

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Cooper Nuclear Station										DOCKET NUMBER (2) 0 5 0 0 0 2 9 8 1				PAGE (3) 1 OF 2									
TITLE (4) RCIC Speed Control Failure																							
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)													
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)										
0	4	0	5	8	4	8	4	0	0	4	0	0	4	3	0	8	4	0	5	0	0	0	0
OPERATING MODE (9) N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																					
POWER LEVEL (10) 0 9 5		20.402(b)				20.406(c)				60.73(a)(2)(iv)				73.71(b)									
		20.406(a)(1)(i)				60.36(c)(1)				X 60.73(a)(2)(v)				73.71(e)									
		20.406(a)(1)(ii)				60.36(c)(2)				60.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)									
		20.406(a)(1)(iii)				60.73(a)(2)(ii)				60.73(a)(2)(viii)(A)													
		20.406(a)(1)(iv)				60.73(a)(2)(iii)				60.73(a)(2)(viii)(B)													
		20.406(a)(1)(v)				60.73(a)(2)(iii)				60.73(a)(2)(ix)													
LICENSEE CONTACT FOR THIS LER (12)																							
NAME Ralph Krause, Instrument and Controls Engineer										AREA CODE 4 0 2		TELEPHONE NUMBER 8 2 5 - 3 8 1 1											
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																							
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC													
X	B	N	S	C	W	2	9	0	Y														
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR									
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO													

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While performing a RCIC monthly functional test, the operator observed a gradual increase in turbine speed and pump discharge pressure with a zero controller output. The operator took manual remote speed control and attempted to adjust the speed with the test speed potentiometer, without success. The turbine was tripped manually from the Control Room and RCIC, an engineered safeguard, was declared inoperable. The Ramp Generator and Signal Converter Box in the Turbine Governor Control System was found defective and was replaced. The RCIC system was retested and performed satisfactorily.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED CMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Cooper Nuclear Station	0500029884	-0	104	-0	002	OF 012

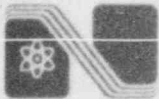
TEXT (If more space is required, use additional NRC Form 366A's) (17)

During normal operation on April 5, 1984, while performing Surveillance Procedure 6.3.6.1 (RCIC Test Mode Surveillance Operation), the operator observed turbine speed and pump discharge pressure slowly increasing with a zero controller output. The operator took manual remote speed control and attempted to adjust the speed with the test speed potentiometer. The speed could not be adjusted, so the turbine was manually tripped from the Control Room. RCIC, an engineered safeguard system, was declared inoperable at 1135 hours. The Ramp Generator and Signal Converter (R.G.S.C.) in the Turbine Governor Control System was replaced and the RCIC system retested satisfactorily. RCIC was declared operable at 1929 hours on April 5, 1984.

The R.G.S.C. is a Woodward Governor Model 8271-083, and supplies a speed reference signal to the turbine speed controller (Woodward Governor Model EG-M Control Box). During automatic startup, the R.G.S.C. provides a linear increasing speed signal to the controller, to bring the turbine from zero to approximately 4250 RPM in a nominal 12 seconds. After the turbine has reached operating speed, the R.G.S.C. will automatically switch from a ramping signal to a pump flow versus turbine speed signal.

An investigation of the defective R.G.S.C. revealed a broken solder joint on a resistor in the signal input circuit. The joint was resoldered and the repaired R.G.S.C. was tested satisfactorily.

During this event, High Pressure Coolant Injection System (HPCI), the redundant system, was operable. This occurrence presented no adverse consequences from the standpoint of public health and safety. No further action is planned.



Nebraska Public Power District

COOPER NUCLEAR STATION
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CNSS840142

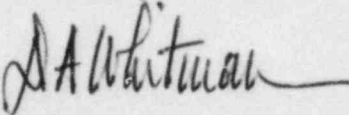
April 30, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Enclosed please find Licensee Event Report No. 84-004 for Cooper Nuclear Station, Docket No. 05000-298. A copy of this report has been submitted to the NRC Regional Office (Region IV) in Arlington, Texas.

Sincerely,

for 

P. V. Thomason
Division Manager of
Nuclear Operations

PVT:lb

cc: J. T. Collins
L. G. Kuncel
J. M. Pilant
INPO Records Center

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