

ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14604

May 30, 1973

Mr. James P. O'Reilly, Director
U.S. Atomic Energy Commission
Region 1
Directorate of Regulatory Operations
970 Broad Street
Newark, New Jersey 07102

Subject: R.E. Ginna Nuclear Power Plant, Unit No. 1
Docket No. 50-244

Dear Mr. O'Reilly:

We have now completed an engineering review, as outlined in our letter of November 1, 1972, of all valves important to nuclear safety installed at Ginna Station. This review represents the first phase of our program to demonstrate acceptable wall thickness on valves important to nuclear safety as required by your letter of June 22, 1972.

We have identified 55 valves over 1-inch nominal pipe size which are within the reactor coolant pressure boundary. The pressure rating of each valve as specified in the applicable code or standard has been compared with the system design pressure. We have found that none of these valves has a pressure rating significantly higher than required by the design conditions.

We, therefore, will perform physical or ultrasonic measurement of wall thickness on all the valves described above. Measurements will begin during our next scheduled refueling shutdown in the spring of 1974. We plan to complete this program by June 22, 1977.

Very truly yours,

Keith W. Amish
Keith W. Amish

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