

MONTHLY OPERATIONS REPORT

MARCH 1984

Throughout the report period, the Oyster Creek Station remained shutdown with the reactor vessel defueled for the Maintenance and Refueling Outage.

The reactor head was lifted and restored on the 119' level and on March 3, 1984, the reactor cavity flooding started. The flooding of the cavity was completed on March 4, 1984 and the fuel pool gates were opened on March 7, 1984. A three-day delay in opening the gates was permitted to allow fuel pool cleanup work to continue and not be jeopardized by the poor water clarity in the cavity. The reactor steam separator was stored in the equipment pool.

LPRM replacement was restarted and an additional six (6) LPRMs were identified as requiring replacement during the report period by Plant Engineering. Stuck LPRM 28-41 was finally removed and replaced. However, the plant had no success in installing a new LPRM in housing 20-49. At the end of the report period, five (5) LPRMs were left to be replaced (including LPRM 20-49).

At the end of the report period, the Operations Department was performing valve lineups as systems became available in preparation for reactor refueling. Installation of Cleanup System isolation valves V-16-1 and V-16-2 was also in progress.

Channelling of the irradiated fuel assemblies (40 total) was also in progress in preparation for reactor refueling. The channelling of five (5) fuel assemblies has been completed. Preventive maintenance was also performed on the refueling bridge. This was a prerequisite to channelling.

On March 3, 1984, the annual inspection of Diesel Generator No. 2 was completed and the unit was returned to service. Subsequently on March 20, 1984 diesel generator was removed from service in order to support plant electrical system preventive maintenance which was conducted over the next five consecutive days. And on March 27, 1984, the unit was temporarily taken out of service in order to make a wiring correction.

Diesel Generator No. 1 was removed from an operable status on March 6, 1984 for preventive maintenance; however, the results of the following start test were unsatisfactory. The problem was attributed to the oil booster pump which was replaced. The diesel generator was returned to service on March 10, 1984. Afterward Diesel Generator No. 1 was again removed from operation on March 15, 1984 for testing and on March 26, 1984 for wire verification. During the period of time each diesel generator was out of service its associated Standby Gas Treatment System (SGTS) was considered inoperable. Due to these restrictions, 119' level activity in the Reactor Building was limited in accordance with Technical Specifications.

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Both Standby Gas Treatment Systems (SGTS) were declared inoperable on March 19, 1984 due to indicated high system flow rates. A subsequent investigation revealed the problem, which was identified as flow interference between the annubar and pitot tube flow sensors, did not impact on either systems' ability to properly operate. The problem was corrected and both SGTS were returned to service the same day.

An inspection of the SRM and IRM dry tubes (nuclear instrumentation) revealed damage to seven (7) IRM dry tubes and one (1) SRM dry tube. Plant Engineering is evaluating the integrity of the nuclear instrumentation from an end-of-life standpoint. New dry tubes are expected to arrive on site in mid-April, and as a minimum, the damaged dry tubes will be replaced.

During a hydro-test of "A" isolation condenser, a leak in the condensate return line was discovered. Insulation on the line was removed and the piping was drained to facilitate non-destructive examination (NDE) of the piping. This inspection was in progress at the end of the report period.

The overhaul of "B" CRD pump was near completion at the end of the month. Alignment and coupling of the pump remained to be completed.

Repairs to Condensate System valves was completed and the Condensate System ("C" pump) was placed in service on March 28, 1984.

A leak in the Old Radwaste overboard discharge line was identified on March 14, 1984. Efforts are presently in progress to uncover the pipe and make repairs. Results of soil/water samples showed activity levels well below 10 CFR limits. No environmental concerns resulted from this event.

On March 1, 1984, a fuel bundle was dropped into a spent fuel rack. The fuel assembly was approximately three feet into the spent fuel rack when the fuel assembly dropped. The bundle was not damaged. The subsequent investigation determined the cause to be operator error. To prevent recurrence, a CCTV camera has been installed and positioned on the grapple in a manner that provides visual verification of positive fuel bail handle engagement.

During backwashing of the fuel pool filter, excessive air leakage into the fuel pool system caused water to enter the ventilation system ducts on the Reactor Building 119' level. This caused contamination of the 75', 51' and 23' elevations. Plant personnel immediately started to reclaim the areas affected. A critique of the incident was performed, and the cause was identified to be leaky valves in this particular valve lineup.

Inspection of the feedwater heaters and piping continued during the report period. Inspection revealed that extensive repairs are required on the heaters and the piping.

Two more operators resumed licensing duties on March 30, 1984. This leaves only three participants remaining in the Accelerated Requalification Program.

The M&C Department investigated the failure of core spray system air operated check valve NZ02-C but could not identify a direct cause for failure of the valve to initially stroke satisfactorily. The valve was disassembled, inspected and the "O" rings on the air actuator were replaced. The valve operated properly during subsequent testing.

Technical Functions/Plant Engineering reached the decision that the leak in the Cleanup System regenerative heat exchanger would not adversely affect the system's operability. Therefore, no repairs are planned.

At the end of the report period, replacement of the expansion joint in "B" main condenser and the cleanout of "B" hotwell were in the final closeout stages. "A" main condenser expansion joint replacement and the cleanout of its hotwell is scheduled next.

Although O.C. was not specifically named, on March 24, 1984 the JCP&L Dispatcher in Allenhurst received what could have been considered a bomb threat. Furthermore, a second bomb threat was received by the plant on March 31, 1984. In both instances, the appropriate actions were taken and proper notifications made.

There were no Licensee Event Reports submitted during March 1984.

1091W

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH March 1984

DOCKET NO. 50-219
UNIT NAME Oyster Creek
DATE 4/3/84
COMPLETED BY R. Baran
TELEPHONE 971-4640

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
31	2-11-83	S	9936	C	1	N/A	ZZ	ZZZZZZ	Start of the 1983 Refueling and Maintenance Outage.

¹
F: Forced
S: Scheduled

²
Reason:
A-Equipment Failure (Explain)
B-Maintenance of Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

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Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

⁴
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-
0161)

⁵
Exhibit I - Same Source

OPERATING DATA REPORT
OPERATING STATUS

1. DOCKET: 50-219
2. REPORTING PERIOD: March, 1984
3. UTILITY CONTACT: JOSEPH R. MOLNAR 609-971-4699
4. LICENSED THERMAL POWER (MWt): 1930
5. NAMEPLATE RATING (GROSS MWe): $687.5 \times 0.8 = 550$
6. DESIGN ELECTRICAL RATING (NET MWe): 650
7. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 650
8. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 620
9. IF CHANGES OCCUR ABOVE SINCE LAST REPORT, GIVE REASONS: NONE
10. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): N/A
11. REASON FOR RESTRICTION, IF ANY: NONE

	<u>MONTH</u>	<u>YEAR</u>	<u>CUMULATIVE</u>
12. REPORT PERIOD HRS	744.0	2184.0	125112.0
13. HOURS RX CRITICAL	0.0	0.0	84623.9
14. RX RESERVE SHUTDOWN HRS	0.0	0.0	468.2
15. HRS GENERATOR ON-LINE	0.0	0.0	82693.8
16. UT RESERVE SHUTDOWN HRS	0.0	0.0	0.0
17. GROSS THERM ENER (MWH)	0.0	0.0	136224729
18. GROSS ELEC ENER (MWH)	0.0	0.0	46056905
19. NET ELEC ENER (MWH)	-1830	-5716	44279967
20. UT SERVICE FACTOR	0.0	0.0	66.1
21. UT AVAIL FACTOR	0.0	0.0	66.1
22. UT CAP FACTOR (MDC NET)	0.0	-0.4	57.1
23. UT CAP FACTOR (DER NET)	0.0	-0.4	54.4
24. UT FORCED OUTAGE RATE	0.0	0.0	9.7
25. FORCED OUTAGE HRS	0.0	0.0	8916.8
26. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, DURATION):	N/A		
27. IF CURRENTLY SHUTDOWN ESTIMATED STARTUP TIME:	6/11/84		

AVERAGE DAILY POWER LEVEL
NET MWe

DOCKET #.50-219
UNIT.Oyster Creek #1
REPORT DATEAPRIL 9, 1984
COMPILED BYDONALD V. NOTIGAN
TELEPHONE #609-971-4695

MONTH MARCH, 1984

<u>DAY</u>	<u>MW</u>	<u>DAY</u>	<u>MW</u>
1.	0	16.	0
2.	0	17.	0
3.	0	18.	0
4.	0	19.	0
5.	0	20.	0
6.	0	21.	0
7.	0	22.	0
8.	0	23.	0
9.	0	24.	0
10.	0	25.	0
11.	0	26.	0
12.	0	27.	0
13.	0	28.	0
14.	0	29.	0
15.	0	30.	0
		31.	0

REFUELING INFORMATION - March, 1984

Name of Facility: Oyster Creek Station #1

Scheduled date for next refueling shutdown: Presently shutdown for Refueling

Scheduled date for restart following refueling: June 11, 1984

Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

Technical Specification Change Request No. 96 was submitted on August 31, 1982 for incorporation of GE assemblies into the Cycle 10 core.

Scheduled date(s) for submitting proposed licensing action and supporting information:

October 28, 1983 - The final supplement to the reload analysis, delineating the specific core configuration for Cycle 10 operation, was submitted.

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

1. General Electric Fuel Assemblies - fuel design and performance analysis methods have been approved by the NRC. New operating procedures, if necessary, will be submitted at a later date.
2. Exxon Fuel Assemblies - no major changes have been made nor are there any anticipated.

The number of fuel assemblies (a) in the core = 0
(b) in the spent fuel storage pool = 1415

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present: 1,800

Planned: 2,600

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Full core offload capability will be lost after the 1985 outage. Batch discharge capability will be lost after the 1987 outage. Expanded spent fuel pool rack capacity (2,600) is scheduled for 1984.



GPU Nuclear Corporation

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Forked River, New Jersey 08731-0388
609 971-4000
Writer's Direct Dial Number.

April 13, 1984

Director
Office of Management Information
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Monthly Operating Report

In accordance with the Oyster Creek Nuclear Generating Station Operating License No. DPR-16, Appendix A, Section 6.9.1.C, enclosed are two (2) copies of the Monthly Operating Data (gray book information) for the Oyster Creek Nuclear Generating Station.

If you should have any questions, please contact Mr. Michael Laggart at (609) 971-4643.

Very truly yours,

Peter B. Fiedler
Vice President and Director
Oyster Creek

PBF:dam
Enclosures

cc: Director (10)
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dr. Thomas E. Murley, Administrator
Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
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NRC Resident Inspector
Oyster Creek Nuclear Generating Station
Forked River, NJ 08731

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