NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
LaSalle County Station Unit 1	0 5 0 0 0 3 7 3	1 OF 0 2

TITLE (4)
Reactor Building Vent Isolation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 3	2 7	8 4	8 4	0 2 0	0 0	0 4	2 3	8 4	NA	0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 8 0	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(a)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(a)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Daniel R. Smythe, extension 292	AREA CODE 8 1 5 — 3 5 7 - 6 7 6 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	L L	R I S	G 0 8 0	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Unit 1 was operating at 80% power. 'D' Reactor Building Vent Process Rad Monitor (IL) was in the tripped condition with no alarms present. During performance of Instrument Maintenance surveillance LIS-VR-01, 'C' Reactor Building Vent (VA) PRM was placed in the tripped condition causing a Group IV Primary Containment Isolation System Isolation (JM).

Per LOA-VR-01, the Group IV PCIS Isolation was reset. The PCIS Group I Isolation was bypassed to prevent a spurious Main Steam tunnel differential temperature trip, while the Reactor Building HVAC (VA) System was restarted. The PCIS Group I Isolation bypass was then removed.

The 'D' Reactor Building Vent PRM was repaired by replacing the relay board inside the trip unit.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

I. EVENT DESCRIPTION

On 3/27/84 at 0244, during performance of LIS-VR-01, 'C' Reactor Building Vent PRM (IL) was placed in the tripped condition. Due to 'D' Reactor Building Vent PRM also being in the tripped condition, unknown at the time, a Group IV isolation was received. The trip in 'C' Reactor Building Vent PRM was immediately reset.

At 0250, per LOA-VR-01, the PCIS Group I isolation trip logic was bypassed to prevent spurious Main Steam tunnel high differential temperature trips while the Reactor Building HVAC System was restarted. At 0335, the PCIS Group I isolation bypass was removed, within the 1 hour limit for restoring the minimum number of channels required before manually tripping one channel and taking Action 21 of Technical Specification Table 3.3.2-1.

II. CAUSE

Due to a failure within the trip unit relay board on 'D' Reactor Building Vent PRM, a high rad trip was present without the associated Control Room annunciator or local trip lignt energized. When 'C' Reactor Building Vent was placed in the "TRIP CHECK" mode, with 'D' Reactor Building Vent PRM in the tripped condition, an unexpected isolation occurred.

III. PROBABLE CONSEQUENCES OF THE OCCURRENCE

The failure was in the conservative direction in that the 'D' Reactor Building Vent PRM failure caused a Group IV PCIS isolation. The Standby Gas Treatment (BH) System auto started upon the Group IV isolation and a negative pressure of greater than or equal to 0.25 inches water column was maintained in the Reactor Building, preventing uncontrolled radioactive releases to the environment.

IV. CORRECTIVE ACTION

The 'D' Reactor Building Vent PRM relay circuit board was replaced. The PRM was verified functional per performance of LIS-VR-01 functional test.

V. PREVIOUS OCCURRENCES

A similar event where a Group IV isolation occurred during LIS-VR-01 (broken lead in trip circuit) happened in April, 1983 (DVR 1-1-83-156).

VI. NAME AND TELEPHONE NUMBER OF PREPARER

Daniel R. Smythe, extension 292.

Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
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Telephone 815/357-6761

April 23, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Reportable Occurrence Report #84-020-00, Docket #050-373 is being submitted to your office in accordance with 10 CFR 50.73.

G. J. Diederich
Superintendent
LaSalle County Station

GJD/MLD/kg

Enclosure

xc: NRC, Regional Director
INPO-Records Center
File/NRC

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