NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1	LICENSEE CODE: W I P B H 1 (2)	LICENSE NUMBER: 0 0 - 0 0 0 0 0 - 0 0 (3)	LICENSE TYPE: 4 1 1 1 1 (4)	CAT: (5)

CON'T

0 1	REPORT SOURCE: L (6)	DOCKET NUMBER: 0 5 0 0 0 2 6 6 (7)	EVENT DATE: 1 0 0 1 8 3 (8)	REPORT DATE: 0 4 2 3 8 4 (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 On 10/01/83, Unit 1 was shut down for refueling and SG replacement. Type
0 3 "B" & "C" leakage tests were performed during the outage, and on 10/07/83
0 4 the total as-found leakage exceeded the TS (15.4.4.II.B & III.B) limit of
0 5 0.6 La because of high leakage through the charging line check valve
0 6 (1-370) and a purge supply valve (1CV-3245). This event is reportable
0 7 in accordance with TS 15.6.9.2.A.3 and is similar to LER's 83-007 and
0 8 82-020.

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP SUBCODE	VALVE SUBCODE
0 9	S D (11)	E (12)	B (13)	V A L V E X (14)	B (15)	D (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 3	—	0 0 9	/	0 1	T	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
B (18)	Z (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	Y (24)	A (25)	F 1 3 5 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Both valves did not seat properly due to foreign material on the valves
1 1 seating surfaces. 1-370 is a 3" 1500 lb SS Velan check valve and 1CV-3245
1 2 is a 36" 9200 series T-ring Fisher butterfly valve. Each valve was
1 3 cleaned, inspected, maintained & successfully retested. After maintenance
1 4 the total Type "B" and "C" leakage was 12031 sccm or 5.2% of 0.6 La.

1 5	FACILITY STATUS: H (28)	% POWER: 0 0 0 (29)	OTHER STATUS (30): N/A	METHOD OF DISCOVERY: B (31)	DISCOVERY DESCRIPTION (32): Surveillance testing

1 6	ACTIVITY RELEASED: Z (33)	CONTENT OF RELEASE: Z (34)	AMOUNT OF ACTIVITY (35): N/A	LOCATION OF RELEASE (36): N/A

PERSONNEL EXPOSURES

1 7	NUMBER: 0 0 0 (37)	TYPE: Z (38)	DESCRIPTION (39): N/A

PERSONNEL INJURIES

1 8	NUMBER: 0 0 0 (40)	DESCRIPTION (41): N/A

LOSS OF OR DAMAGE TO FACILITY (43)

1 9	TYPE: Z (42)	DESCRIPTION: N/A

PUBLICITY

2 0	ISSUED: N (44)	DESCRIPTION (45): N/A	NRC USE ONLY

NAME OF PREPARER C. W. Fay PHONE: 414/277-2811

ATTACHMENT TO LICENSEE EVENT REPORT NO. 83-009/01T-0

Wisconsin Electric Power Company
Point Beach Nuclear Plant Unit 1
Docket No. 50-266

On October 1, 1983, Unit 1 was removed from service for its eleventh annual refueling and steam generator replacement. On October 7, 1983, an initial review of the Type "B" and "C" leakage tests of containment isolation devices revealed that two valves had leakage which, when combined, exceeded the limit in Technical Specification 15.4.4.3.B. In addition, it was also discovered that a third valve had experienced an internal component failure which affected the validity of the associated leakage test. The above-mentioned valves include the containment purge supply valve (1CV-3245), the charging line check valve (1-370) and the containment purge exhaust valve (1CV-3212).

The containment purge supply and exhaust valves are 36", 9200 series, "T" ring Fisher governor butterfly valves. Please note that the 24-hour written notification reported that these valves were manufactured by "Miller." This is an error as the valves are actually Fisher governor valves. The pneumatic control system for the Fisher governor valve contains "Miller" valves.

On October 1, 1983, the purge supply valve was leak tested. Pressurization to the required test pressure could not be obtained (i.e., 60 psig). The valve was found to leak at 105 slpm at a maximum obtainable pressure of 63.6 psia. Initial investigation into the problem indicated the valve was not properly seated. The valve disc was manually adjusted, resulting in a leakage rate of 263 sccm at a test pressure of 80.2 psia. Followup maintenance action revealed that an accumulation of dirt on the valve's sealing surfaces prevented it from seating properly. No problems were identified in the valves closing/sealing subsystem. The valve was cleaned, retested and returned to service. Subsequent leakage testing, performed on April 3, 1984, (following a Type "A" containment integrated leak rate test, but prior to unit startup) revealed an insignificant leakage of 10 sccm at test pressure.

The charging line check valve (1-370) is a 3", 1500 lb, stainless steel, Velan swinging disc check valve. This valve was initially leak tested on October 7, 1983. Pressurization to the required test pressure could not be achieved. The valve was found to leak at 176 slpm at the maximum obtainable pressure of 46 psia. Followup maintenance action revealed that there were brass chips within the valve, preventing proper seating. The valve was cleaned, lapped and reassembled. Followup leakage testing revealed an insignificant leakage of 14 sccm at test pressure.

On October 1, 1983, the purge exhaust valve was also tested. This valve was found to leak at 470 sccm at a test pressure of 84.25 psia. However, the validity of this test is questionable as the inflatable boot incorporated into the valve was found leaking into the test volume. It should be noted that this condition did not significantly affect the performance of the valve as evidenced by achieving test pressure with a low flow rate through the test apparatus. The subject valve was repaired with a new inflatable seal, successfully retested and returned to service. Subsequent leakage testing, performed on April 4, 1984, revealed a very low leakage rate of 224 sccm at test pressure.

The containment purge supply and exhaust penetrations both contain redundant containment isolation valves. The charging line penetration contains manual containment isolation valves in addition to the check valve, and is also a closed system outside of containment. Thus, a backup was available to maintain the overall leak tightness of the Unit 1 containment. These events had no effect on the public health and safety.

The total Type "B' and "C" leakage testing program for the Unit 1 Refueling 11 was completed on April 4, 1984. The total as-found leakage, excluding valve 1-370, 1CV-3212 and 1CV-3245 was 27,722 sccm or 12.0% of allowable. The total leakage after maintenance of the subject valves, and several other valves with greater than desirable leakage was 12,031 sccm or 5.2% of allowable.

This event is reportable in accordance with Technical Specification 15.6.9.2.A.3, "Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment. The Resident Inspector has been notified of this event.

DMB

Wisconsin Electric POWER COMPANY
231 W. MICHIGAN, P.O. BOX 2046, MILWAUKEE, WI 53201

April 23, 1984

Mr. J. G. Keppler, Regional Administrator
Office of Inspection and Enforcement,
Region III
U. S. NUCLEAR REGULATORY COMMISSION
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Dear Mr. Keppler:

DOCKET NO. 50-266
LICENSEE EVENT REPORT NO. 83-009/01T-0
POINT BEACH NUCLEAR PLANT, UNIT 1

Enclosed is Licensee Event Report No. 83-009/01T-0 with an attachment which provides a description of an event reportable in accordance with Technical Specification 15.6.9.2.A.3, "Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment."

The initiating event for this report occurred on October 1, 1983, however, submittal of this report was delayed until all Unit 1 Refueling 11 Type "B" and "C" testing data were available. The Type "B" and "C" testing was completed on April 4, 1984.

Very truly yours,

Vice President-Nuclear Power

C. W. Fay

Copy to NRC Resident Inspector

APR 26 1984

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