

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Nine Mile Point Unit #1										DOCKET NUMBER (2) 0 5 0 0 0 2 2 0				PAGE (3) 1 OF 0 2			
TITLE (4) Control Rod Drive Housing Leak																	
EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)								
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)				
0	3	2	7	8	4	8	4	0	0	2	0	0	0	0	0		
0	3	2	7	8	4	8	4	0	0	2	0	0	0	0	0		
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)															
N		20.402(b)				20.406(e)				80.73(a)(2)(iv)				73.71(b)			
POWER LEVEL (10)		0 0 0				20.406(a)(1)(i)				80.73(a)(2)(v)				73.71(c)			
		20.406(a)(1)(ii)				50.36(e)(1)				80.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)			
		20.406(a)(1)(iii)				50.36(e)(2)				80.73(a)(2)(viii)(A)							
		20.406(a)(1)(iv)				50.73(a)(2)(i)				80.73(a)(2)(viii)(B)							
		20.406(a)(1)(v)				50.73(a)(2)(ii)				80.73(a)(2)(ix)							
LICENSEE CONTACT FOR THIS LER (12)																	
NAME Fred Hawksley										TELEPHONE NUMBER AREA CODE 3 1 5 3 4 9 - 2 4 6 1							
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																	
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS							
B	A	A R P V	C	4 9 0	Yes												
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR	
YES (If yes, complete EXPECTED SUBMISSION DATE)												NO					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

During an inspection of the control rod drive housings, a small leak was discovered at one location, and evidence of leakage was discovered at other locations. The cause is believed to be intergranular stress corrosion cracking (IGSCC) in the stub tube heat affected zone of the stub tube to control rod drive housing welds.

Corrective action taken was to roll these housings into the reactor vessel penetration.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Nine Mile Point Unit #1	0 5 0 0 0 2 2 0 8 4 --	0 0	2 --	0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

NARRATIVE

On March 27, 1984, at approximately 10:30 am, during a refueling outage, an inspection was performed on the control rod drive (CRD) housings. At location 46-27, a small leak (approximately 15 drops per minute) was observed at the CRD housing flange. At location 14-11, water bead marks were observed at the top of the CRD housing flange. Additionally, there was either slight corrosion or film on the CRD housing at locations; 30-07, 34-07, 38-31, 42-39, 34-39, 10-27 and 46-39.

The cause of this event is attributed to intergranular stress corrosion cracking, which allowed the water to penetrate to the outside of the reactor vessel.

ASSESSMENT OF SAFETY CONSEQUENCES

Potential consequences resulting from this incident have been analyzed by General Electric and include control rod drive ejection and a loss of coolant accident. It has been determined that a 360° through wall crack would not result in a rod ejection and the tight tolerance through the control rod drive assembly would not allow a significant amount of coolant to penetrate out of the reactor vessel.

CORRECTIVE ACTION

Control rod drive housings that showed evidence of leaking, along with the housing that leaked, have been rolled, ultrasonic tested, and will be inspected during the pre-startup hydrostatic test.

NIAGARA MOHAWK POWER CORPORATION

NIAGARA  MOHAWK

300 ERIE BOULEVARD, WEST
SYRACUSE, N. Y. 13202

April 26, 1984

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

RE: Docket No. 50-220
LER 84-02

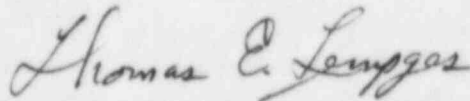
Gentlemen:

In accordance with 10 CFR 50.73, we hereby submit the following
Licensee Event Report:

LER 84-02 Which is being submitted in accordance with
10 CFR 50.73 (b)(II), "Any event or condition
that resulted in the condition of the Nuclear
Power Plant, including its principal safety
barrier, being seriously degraded"

The report was completed in the format designated in NUREG-1022,
dated September 1983.

Very truly yours,



Thomas E. Lempges
Vice President
Nuclear Generation

TEL/lo
Attachment (3 copies)
cc: Dr. Thomas E. Murley
Regional Administrator

IE22
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