

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3										DOCKET NUMBER (2) 0 5 0 0 0 3 6 2				PAGE (3) 1 OF 0 6		
TITLE (4) DOSE EQUIVALENT IODINE LIMITS EXCEEDED																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (3)						
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
0 3	3 0	8 4	8 4	0 1 3	0 0	0 4	2 6	8 4					0 5 0 0 0 3 6 2			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)														
3		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)		
POWER LEVEL (10)		20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)		
0 0 0		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				X OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)						
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)						
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME J. G. HAYNES, STATION MANAGER										TELEPHONE NUMBER AREA CODE 7 1 4 4 9 2 - 7 7 0 0						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC						
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)												X NO				

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Pursuant to Limiting Condition for Operation (LCO) 3.4.7, Action Statement 'd' of Appendix A, Technical Specifications to Facility Operating License NPF-15 for San Onofre Unit 3, this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving the Reactor Coolant System specific activity.

On 3/30/84, at 1900, with Unit 3 in Mode 3 following a reactor shutdown for maintenance, a Reactor Coolant System (RCS) sample analysis indicated that RCS specific activity exceeded 1.0 microcurie/gram Dose Equivalent (DE) I-131. Purification flow was increased and RCS activity was reduced to less than 1.0 microcurie/gram DE I-131 at 0020 on 4/1/84.

On 4/1/84, at 0905, with Unit 3 in Mode 2 at 5% power, RCS sample analysis again indicated that RCS activity exceeded 1.0 microcurie/gram Dose Equivalent (DE) I-131. Purification flow was increased and RCS activity was reduced to less than 1.0 microcurie/gram DE I-131 at 1310 on 4/1/84. No further corrective action is planned.

8405010385 840426
PDR ADDCK 05000362
S PDR

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3	DOCKET NUMBER (2) 0 5 0 0 0 3 6 2	LER NUMBER (6)			PAGE (3)		
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Pursuant to Limiting Condition for Operation (LCO) 3.4.7, Action Statement 'd' of Appendix A, Technical Specifications to Facility Operating License NPF-15 for San Onofre Unit 3, this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving the Reactor Coolant System specific activity.

On March 30, 1984, at 0300, the Unit 3 reactor was shutdown for a planned maintenance outage. At 1900, analysis of a Reactor Coolant System (RCS) sample indicated that the specific activity exceeded 1.0 microcurie/gram Dose Equivalent (DE) I-131. As corrective action, purification flow was increased. The RCS specific activity was reduced to less than 1.0 microcurie/gram DE I-131 at 0020 on April 1, 1984.

On April 1, 1984, at 0905, analysis of a Reactor Coolant System (RCS) sample again indicated that RCS specific activity exceeded 1.0 microcurie/gram Dose Equivalent (DE) I-131. Purification flow was increased and RCS specific activity was reduced to less than 1.0 microcurie/gram DE I-131 at 1310 on April 1, 1984.

The two events were indications of iodine spiking following a power change. Similar occurrences were previously reported in LER 83-111 and LER 84-005. NSSS Engineering is monitoring and evaluating reactor coolant activity for trends that may be indicative of any significant changes in the fuel integrity. No further corrective action is planned. There was no impact on the health and safety of the public associated with these events.

Additional information required by LCO 3.4.7, Action Statement 'd', is provided on the following pages. Information concerning de-gassing operation is not applicable since this operation is not provided for in the plant design.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

CLEANUP FLOW HISTORY

<u>PERIOD</u>	<u>AVERAGE CLEANUP FLOW (GPM)</u>
3/28/84, 1900 to 3/30/84, 0900	82.65
3/30/84, 1000 to 3/31/84, 1600	85*
3/31/84, 1700 to 4/01/84, 1400	79.98

*Hourly cleanup flow data not available. Figure used is taken from average flow with two charging pumps in operation.

REACTOR POWER HISTORY

<u>PERIOD</u>	<u>REACTOR POWER</u>
3/28/84, 1900 to 3/29/84, 0200	100% Rated Power
3/29/84, 0300 to 3/29/84, 0500	90%
3/29/84, 0600 to 3/29/84, 1900	100%
3/29/84, 2000 to 3/29/84, 2100	90%
3/29/84, 2200	76%
3/29/84, 2300	63.9%
3/30/84, 2400	57.2%
3/30/84, 0100	33.79%
3/30/84, 0200	30.84%
3/30/84, 0300	Reactor Tripped
3/30/84, 0300 to 3/30/84, 1900	0%
3/30/84, 2000 to 3/31/84, 2300	0%
4/01/84, 2400 to 4/01/84, 0100	0.1%
4/01/84, 0200	0.2%
4/01/84, 0300 to 4/01/84, 0700	0.3%
4/01/84, 0800	0.2%
4/01/84, 0900	0.5%

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

REACTOR COOLANT SYSTEM SPECIFIC ACTIVITY ANALYSIS

<u>DATE</u>	<u>TIME</u>	<u>DE I-131 MICROCURIES/GRAM</u>
3/30/84	1900	1.778
3/30/84	2030	1.741
3/31/84	0035	1.837
3/31/84	0430	1.875
3/31/84	0840	1.855
3/31/84	1230	1.908
3/31/84	1600	1.580
3/31/84	2000	1.276
3/31/84	2220	1.06
4/01/84	0020	0.91
4/01/84	0905	1.061
4/01/84	1310	0.509

The total time with the DE I-131 above 1.0 microcuries/gram for these events was 35.75 hours. Cumulative time with DE I-131 above 1.0 microcuries/gram for last 12-month period is 77.94 hours.

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SAN ONOFRE NUCLEAR GENERATING STATION,
UNIT 3

YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	PAGE (4)	
06000036284	013	00	05	06

TEXT OF EVENT SHOULD BE REPRODUCED, USE ADDITIONAL NRC Form 200A (11/77)

DATE OF EVENT: 06/12/88

TIME OF EVENT: 14:00

REPORTING OFFICE: NRC

REPORTING PERSON: [Name]

DESCRIPTION OF EVENT: [Detailed description of the event, including the time, location, and the actions taken to resolve the issue. The text is organized into paragraphs, with some sections starting with 'The following information was obtained from...' and others starting with 'The following information was obtained from...']

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED ONS NO 3185-0104

EXPIRES 8/31/85

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SAN ONOFRE NUCLEAR GENERATING STATION,
UNIT 3

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NOTE: If more space is required, use additional NRC Form 885a (11)

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09-07-85
10-07-85
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12-07-8501-08-86
02-08-86
03-08-86
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06-08-8607-08-86
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12-02-9201-03-93
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06-03-9307-03-93
08-03-93
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12-03-93ALUMINUM INTERLOCK ASSEMBLY EXPOSURE IS 2.268350E-08 R/MT IN ASSEMBLY 189
ALUMINUM PERMANENTIAL EXPOSURE IS 2.30440E-08 R/MT OCCURRING AT 36.39 P/B OF THE CORE HEIGHT IN ASSEMBLY 189
RE-ARMED EXPOSURE IS 2.21840E-08 R/MT

Equal to 57.064 EFFD

BATCH NUMBER BATCH NAME AVERAGE EXPOSURE RATE
N/A N/A
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Southern California Edison Company



SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES
STATION MANAGER

April 26, 1984

TELEPHONE
(714) 492-7700

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-362
30-Day Report
Licensee Event Report No. 84-013
San Onofre Nuclear Generating Station, Unit 3

Pursuant to Limiting Condition for Operation (LCO) 3.4.7, Action Statement 'd' of Appendix A, Technical Specifications to Facility Operating License NPF-15 for San Onofre Unit 3, this submittal provides the required 30-day written Licensee Event Report (LER) for two occurrences involving the Reactor Coolant System specific activity.

If you require any additional information, please so advise.

Sincerely,

Enclosure: LER No. 84-013

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, NRC Region V)

Institute of Nuclear Power Operations (INPO)

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