

File

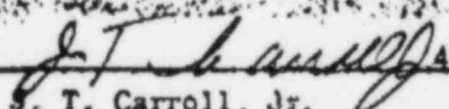
To: James P. O'Reilly
Directorate of Regulatory Operations
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

From: Jersey Central Power & Light Company
Oyster Creek Nuclear Generating Station Docket #50-219
Forked River, New Jersey 08731

Subject: Abnormal Occurrence Report No. 50-219/74/ 24

The following is a preliminary report being submitted
in compliance with the Technical Specifications
paragraph 6.6.2.

Preliminary Approval:


S. T. Carroll, Jr.

4/10/74
Date

cc: Mr. A. Giambusso

8304110050 740410
PDR ADOCK 05000219
S PDR

Handwritten:
50-219

Initial Written
Report Date:

4/10/74

Time of
Occurrence:

1000

OYSTER CREEK NUCLEAR GENERATING STATION
PORKED RIVER, NEW JERSEY 08731

Abnormal Occurrence
Report No. 50-219/74/24

IDENTIFICATION
OF OCCURRENCE:

Violation of the Technical Specifications, paragraph 4.6.B.1.g,
in that the stack gas particulate filter in service from
March 28, 1974 to March 31, 1974 was not analyzed for gross B,
gross α , and gross γ .

This event is considered to be an abnormal occurrence as de-
fined in the Technical Specifications, paragraph 1.15G.

CONDITIONS PRIOR
TO OCCURRENCE:

<input checked="" type="checkbox"/> Steady State Power	<input type="checkbox"/> Routine Shutdown
<input type="checkbox"/> Hot Standby	<input type="checkbox"/> Operation
<input type="checkbox"/> Cold Shutdown	<input type="checkbox"/> Load Changes During
<input type="checkbox"/> Refueling Shutdown	<input type="checkbox"/> Routine Power Operation
<input type="checkbox"/> Routine Startup	<input type="checkbox"/> Other (Specify)
<input type="checkbox"/> Operation	

The major plant parameters at the time of the event were as
follows:

Power:	Reactor, 1806 MWt
	Electrical, 632 MWe
Flow:	Recirc., 57.2×10^6 lb/hr
	Feedwater, 6.735×10^6 lb/hr
Stack Gas:	35,000 μ Ci/sec

DESCRIPTION
OF OCCURRENCE:

A stack gas particulate filter, installed at 0847 on March 28,
1974 and removed at 0854 on March 31, 1974, was not counted
for gross α , gross B, and gross γ within one week of removal.

APPARENT CAUSE
OF OCCURRENCE:

☐ Design
☐ Manufacture
☐ Installation/
☐ Construction
☐ Operator

☐ Procedure
☐ Unusual Service Condition
☐ Inc. Environmental
☐ Component Failure
☐ Other (Specify)

Counting of the filter 48 hours after removal was not performed by the technician as required. Investigation into the cause is continuing.

ANALYSIS OF
OCCURRENCE:

The safety significance connected with this occurrence is that any unusually large release of particulate activity during this period might not have been recognized until the monthly composite analyses were complete. This is not a likely possibility as a spectrum analysis was performed to measure releases of short-lived isotopes and if unusually large amounts of activity were present, it would have been readily apparent. In addition, a spectrum analysis of the charcoal filter had also been performed and normal quantities of Iodine were found.

CORRECTIVE
ACTION:

The particulate filter was counted nine days after filter removal upon discovery of the abnormal occurrence. The gross α , gross β , and gross γ values were comparable to samples removed before and after this filter. This would indicate that there was little or no change in plant gaseous effluents and that the plant was operating with releases less than 4% of the Technical

Specification limit of 4 $\mu\text{Ci/sec}$ of iodine and particulates
having half-lives >8 days.

Prepared by:

Robert L. Stouckman

Date:

4/10/74

To: James P. O'Reilly
Directorate of Regulatory Operations
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

From: Jersey Central Power & Light Company
Oyster Creek Nuclear Generating Station Docket #50-219
Forked River, New Jersey 08731

Subject: Abnormal Occurrence Report No. 50-219/74/ 25

The following is a preliminary report being submitted
in compliance with the Technical Specifications
paragraph 6.6.2.

Preliminary Approval:

J. T. Carroll, Jr. 4/10/74
J. T. Carroll, Jr. Date

cc: Mr. A. Giambusso

Dofe
8304110040

Report Date: 4/9/74Occurrence: 4/9/74

Initial Written

Report Date: 4/10/74

Time of

Occurrence: 1040OYSTER CREEK NUCLEAR GENERATING STATION
FORKED RIVER, NEW JERSEY 08731Abnormal Occurrence
Report No. 50-219/74/25IDENTIFICATION
OF OCCURRENCE:

Violation of the Technical Specifications, paragraph 3.5.A.1, loss of primary containment integrity with the reactor critical and the reactor water temperature greater than 212°F.

This event is considered to be an abnormal occurrence as defined in the Technical Specifications, paragraph 1.15B.

CONDITIONS PRIOR
TO OCCURRENCE:

<input checked="" type="checkbox"/> Steady State Power	<input type="checkbox"/> Routine Shutdown
<input type="checkbox"/> Hot Standby	<input type="checkbox"/> Operation
<input type="checkbox"/> Cold Shutdown	<input type="checkbox"/> Load Changes During
<input type="checkbox"/> Refueling Shutdown	<input type="checkbox"/> Routine Power Operation
<input type="checkbox"/> Routine Startup	<input type="checkbox"/> Other (Specify)
<input type="checkbox"/> Operation	

The major plant parameters at the time of the event were as follows:

Power: Reactor, 1849 MWt
Electrical, 642 MWe
Flow: Recirc., 61×10^6 lbm/hr
Feedwater, 6.9×10^6 lbm/hr
Stack Gas: 34,895 μ Ci/sec

DESCRIPTION
OF OCCURRENCE:

At 1040 on April 9, 1974, a local leak rate test on the reactor building to torus vacuum breakers was commenced and it was discovered that pressure could not be placed between V-26-15 and 16, the check and butterfly isolation valves. A check of leakage of drywell atmosphere to the reactor building was made. No leakage was indicated when a plastic bag was taped over the vent pipe. When pressure was again applied,

the bag filled with air indicating that the outside (check) valve was leaking. The check valve was cycled several times and then forced to seat. Air was again admitted between the valves and no leakage was detected through the check valve, but the pressure would not exceed 17.5 psig. This indicated that the butterfly valve, V-26-16, was leaking. Upon inspection of V-26-16, it was found that the valve had not been in the fully closed position. The valve was fully closed manually and a successful leak test was performed.

The butterfly valve, V-26-16, was made inoperable, as permitted by Technical Specification 3.S.A.5.

APPARENT CAUSE
OF OCCURRENCE:

<input type="checkbox"/> Design	<input type="checkbox"/> Procedure
<input type="checkbox"/> Manufacture	<input type="checkbox"/> Unusual Service Condition
<input type="checkbox"/> Installation/	<input type="checkbox"/> Inc. Environmental
<input type="checkbox"/> Construction	<input checked="" type="checkbox"/> Component Failure
<input type="checkbox"/> Operator	<input type="checkbox"/> Other (Specify)

The cause for valves V-26-15 and V-26-16 not to seal properly is not known at this time.

ANALYSIS OF
OCCURRENCE:

An initial analysis of the data indicates that under accident conditions in the drywell (35 psig) the leakage rate through V-26-15 and V-26-16 would have been approximately 20% to 40% of the allowable leakage rate from the primary containment. Additional investigation is continuing.

CORRECTIVE
ACTION:

At 1408, a reactor shutdown was commenced. Both valves were subsequently properly seated, successfully leak tested, and made inoperable. At 1443, the reactor shutdown was terminated and power was increased to the initial value.

FAILURE DATA:

History of torus to reactor building vacuum breakers:

- 11/23/70 - V-26-18 failed to open during operability surveillance
- 12/18/70 - V-26-16 and V-26-18 failed to open during operability surveillance
- 1/12/71 - V-26-18 failed leakage rate test
- 1/13/71 - V-26-18 linkage tightened one turn, passed leakage test but valve would not open
- 1/14/71 - V-26-18 adjusted controller, passed leakage test and passed operability test
- 2/17/71 - V-26-16 and V-26-18 changed seats and both passed leakage tests
- 5/3/73 - V-26-18 failed leakage test. Linkage adjusted and valve passed leakage and operability tests.

Prepared by:

Karl P. G. Fickens

Date:

4/10/74

Jersey Central Power & Light Company



MADISON AVENUE AT PUNCH BOWL ROAD • MORRISTOWN, N. J. 07960 • 201-539-5111

MEMBER OF THE
General  Public Utilities Corporation

July 10, 1974



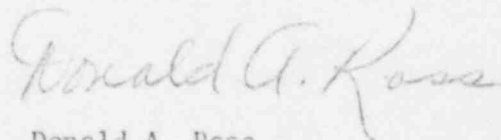
Mr. A. Giambusso
Deputy Director for Reactor Projects
Directorate of Licensing
United States Atomic Energy Commission
Washington, D. C. 20545

Dear Mr. Giambusso:

Subject: Oyster Creek Station
Docket No. 50-219
Stack Gas Filter Cartridge Analysis

In my report of Abnormal Occurrence No. 50-219/74/24, dated April 19, 1974, I indicated that a review of the administrative control over the stack gas filter cartridge analysis would be undertaken to assure that counting is performed on schedule. This review has been completed, and a change in the check-off procedures has been incorporated to insure that the counting schedule is met.

Very truly yours,



Donald A. Ross
Manager, Nuclear Generating Stations

cs

cc: Mr. J. P. O'Reilly, Director
Directorate of Regulatory Operations, Region I

*Received
50-219*

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COPY SENT REGION 1

OYSTER CREEK NUCLEAR GENERATING STATION
FORKED RIVER, NEW JERSEY 08731

Abnormal Occurrence
Report No. 50-219/74/24

Report Date

April 19, 1974

Occurrence Date

April 9, 1974

Identification of Occurrence

Violation of the Technical Specifications, paragraph 4.6.B.1.g, in that the stack gas particulate filter in service from March 28, 1974 to March 31, 1974 was not analyzed for gross β , gross α , and gross γ , but was analyzed for Ba-140, La-140 and I-131 within 48 hours. This event is considered to be an abnormal occurrence as defined in the Technical Specifications, paragraph 1.15G.

Conditions Prior to Occurrence

The plant was operating at steady-state power.

The major plant parameters at the time of the event were as follows (during the period the filter was in service):

Power:	Reactor, 1806 MWt
	Electrical, 632 MWe
Flow:	Recirculation, 57.2×10^6 lb/hr
	Feedwater, 6.735×10^6 lb/hr
Stack Gas:	35,000 μ Ci/sec

Description of Occurrence

A stack gas particulate filter, installed at 0847 on March 28, 1974 and removed at 0854 on March 31, 1974, was not counted for gross β , gross α , and gross γ , as per Technical Specification 4.6.B.1.9. This was detected in a routine audit of the stack release records.

Apparent Cause of Occurrence

Counting of the filter 48 hours after removal was not performed by the technician as required by our normal practice.

Analysis of Occurrence

The safety significance connected with this occurrence is that any unusually large release of particulate activity during this period might not have been

*dupe of
5/13/74*

recognized until the monthly composite analyses were complete. This is not a likely possibility as a spectrum analysis was performed to measure releases of short-lived isotopes and if unusually large amounts of activity were present, it would have been readily apparent. In addition, a spectrum analysis of the charcoal filter had also been performed and normal quantities of iodine were found.

Corrective Action

The particulate filter was counted nine days after filter removal upon discovery of the abnormal occurrence. The gross β , gross α , and gross γ values were comparable to samples removed before and after this filter as is shown in the following table:

<u>Sample Period</u>	<u>Gross α $\mu\text{Ci}/\text{CC} \times 10^{-14}$</u>	<u>Gross β $\mu\text{Ci}/\text{CC} \times 10^{-10}$</u>	<u>Gross γ $\text{CPM}/\text{CC} \times 10^{-5}$</u>
3/22 to 3/25	.487	1.36	2.09
3/25 to 3/28	1.14	1.92	2.84
3/28 to 3/31	.139	1.68	2.11
3/31 to 4/2	2.32	2.81	1.56
4/2 to 4/5	.677	2.13	3.16

This would indicate that there was little or no change in plant stack effluents and that the plant was operating with releases less than 4% of the Technical Specification limit of 4 $\mu\text{Ci}/\text{sec}$ of iodine and particulates having half-lives > 8 days.

The administrative control over stack gas filter cartridge analysis will be reviewed and tightened, if necessary, to assure that counting is performed on schedule.

Jersey Central Power & Light Company



MADISON AVENUE AT PUNCH BOWL ROAD • MORRISTOWN, N. J. 07960 • 201-539-6111

General



Public Utilities Corporation

April 19, 1974

Mr. A. Giambusso
Deputy Director for Reactor Projects
Directorate of Licensing
United States Atomic Energy Commission
Washington, D. C. 20545



Dear Mr. Giambusso:

Subject: Oyster Creek Station
Docket No. 50-219
Abnormal Occurrence Report No. 50-219/74/24

The purpose of this letter is to forward to you an event which was reported as a Technical Specification violation on April 9, 1974. Hindsight leads me to believe this is not a reportable event. Even though the sample was analyzed later than is our normal practice, there was no loss of continuity in the gross counting analysis.

Enclosed are forty copies of this submittal.

Very truly yours,

Donald A. Ross
Manager, Nuclear Generating Stations

DAR/pd
Enclosures

cc: Mr. J. P. O'Reilly, Director
Directorate of Regulatory Operations, Region I

3606
COPY SENT REGION

To: James P. O'Reilly
Directorate of Regulatory Operations
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

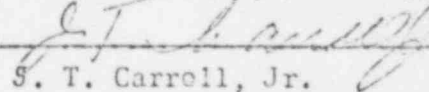


From: Jersey Central Power & Light Company
Oyster Creek Nuclear Generating Station Docket #50-219
Forked River, New Jersey 08731

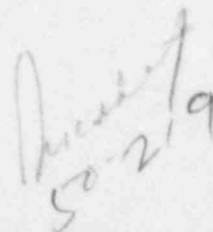
Subject: Abnormal Occurrence Report No. 50-219/74/ 24

The following is a preliminary report being submitted
in compliance with the Technical Specifications
paragraph 6.6.2.

Preliminary Approval:

 4/10/74
S. T. Carroll, Jr. Date

cc: Mr. A. Giambusso


50-219

COPY SENT REGION 2

3332

Initial Telephone
Report Date: _____

Date of
Occurrence: _____ 4/9/74

Initial Written
Report Date: _____ 4/10/74

Time of
Occurrence: _____ 1000

OYSTER CREEK NUCLEAR GENERATING STATION
FORKED RIVER, NEW JERSEY 08731

Abnormal Occurrence
Report No. 50-219/74/24

IDENTIFICATION
OF OCCURRENCE:

Violation of the Technical Specifications, paragraph 4.6.B.1.g,
in that the stack gas particulate filter in service from
March 28, 1974 to March 31, 1974 was not analyzed for gross β ,
gross α , and gross γ .

This event is considered to be an abnormal occurrence as de-
fined in the Technical Specifications, paragraph 1.15G.

CONDITIONS PRIOR
TO OCCURRENCE:

<input checked="" type="checkbox"/> Steady State Power	<input type="checkbox"/> Routine Shutdown
<input type="checkbox"/> Hot Standby	<input type="checkbox"/> Operation
<input type="checkbox"/> Cold Shutdown	<input type="checkbox"/> Load Changes During
<input type="checkbox"/> Refueling Shutdown	<input type="checkbox"/> Routine Power Operation
<input type="checkbox"/> Routine Startup	<input type="checkbox"/> Other (Specify)
<input type="checkbox"/> Operation	

The major plant parameters at the time of the event were as
follows:

Power:	Reactor, 1806 MWt
	Electrical, 632 MWe
Flow:	Recirc., 57.2×10^6 lb/hr
	Feedwater, 6.735×10^6 lb/hr
Stack Gas:	35,000 μ Ci/sec

DESCRIPTION
OF OCCURRENCE:

A stack gas particulate filter, installed at 0847 on March 28,
1974 and removed at 0854 on March 31, 1974, was not counted
for gross α , gross β , and gross γ within one week of removal.

APPARENT CAUSE
OF OCCURRENCE:

☐ Design
☐ Manufacture
☐ Installation/
☐ Construction
☐ Operator

☐ Procedure
☐ Unusual Service Condition
☐ Inc. Environmental
☐ Component Failure
☐ Other (Specify)

Counting of the filter 48 hours after removal was not performed by the technician as required. Investigation into the cause is continuing.

ANALYSIS OF
OCCURRENCE:

The safety significance connected with this occurrence is that any unusually large release of particulate activity during this period might not have been recognized until the monthly composite analyses were complete. This is not a likely possibility as a spectrum analysis was performed to measure releases of short-lived isotopes and if unusually large amounts of activity were present, it would have been readily apparent. In addition, a spectrum analysis of the charcoal filter had also been performed and normal quantities of Iodine were found.

CORRECTIVE
ACTION:

The particulate filter was counted nine days after filter removal upon discovery of the abnormal occurrence. The gross α , gross β , and gross γ values were comparable to samples removed before and after this filter. This would indicate that there was little or no change in plant gaseous effluents and that the plant was operating with releases less than 4% of the Technical

Specification limit of 4 μ Ci/sec of Iodine and particulates
having half-lives >8 days.

Prepared by: *Robert L. Stuchman* Date: 4/10/74