

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Pilgrim Nuclear Power Station (PNPS) - Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 2 9 3				PAGE (3) 1 OF 2					
TITLE (4) Safety Valve Setpoints Below Requirement of Technical Specifications																			
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)						
0	3	28	84	004	00	0	4	26	84					0 5 0 0 0					
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																	
N		20.402(b)				20.408(e)				50.73(a)(2)(iv)				73.71(b)					
POWER LEVEL (10)		20.408(a)(1)(i)				50.38(e)(1)				50.73(a)(2)(v)				73.71(e)					
0 1 0 0		20.408(a)(1)(ii)				50.38(e)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)					
		20.408(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)									
		20.408(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)									
		20.408(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)									
LICENSEE CONTACT FOR THIS LER (12)																			
NAME Paul J. Hamilton - Plant Engineer										TELEPHONE NUMBER									
										AREA CODE									
										6 1 1 7 7 1 4 6 1 - 7 9 1 0 0									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																			
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS									
X	SIB	RIV	D 2 4 3	Y															
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR			
X YES (If yes, complete EXPECTED SUBMISSION DATE)												NO		0	7	1	3	8	4

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 3/28/83, during a refueling outage, the Maintenance Department was notified by Wyle Laboratories that both of the Main Steam Safety Valves exhibited set pressures more than 1% below the nameplate set pressure. This is contrary to the requirements of PNPS Technical Specification (T.S.) 2.2.C, which requires both valves to lift at 1240 psi  $\pm$  13 psi. When tested, one valve lifted at 1209 psi, and the other lifted at 1155 psi.

Cause and corrective action are pending receipt of a written report from Wyle Lab.

8405010263 840426  
PDR ADDCK 05000293  
S PDR

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Pilgrim Nuclear Power Station-Unit 1	0500029384	—	004	—	002	OF 02

TEXT (If more space is required, use additional NRC Form 355A's) (17)

On 3/28/84, during a refueling outage, the Maintenance Department was notified by Wyle Laboratories that both of the PNPS Dresser Main Steam Safety Valves, Model 3777, exhibited set pressures more than 1% below the nameplate set pressure. This is contrary to the requirements of PNPS Technical Specification T.S. 2.2.C, which requires both valves to lift at 1240 psi  $\pm$  13 psi. The subject valves were being tested in accordance with the requirements of T.S. 4.6.D.1.

Both valves were tested twice. Valve #203-4A, Serial #BK6262 lifted at 1213 psi during Test 1 and 1209 psi during Test 2.

Valve #203-4B, Serial #BK6309 lifted at 1165 psi during Test 1 and 1155 psi during Test 2.

Determination of cause and development of a corrective action plan is pending receipt of a report from Wyle Lab. An update report will be submitted after receipt of the Wyle Report.

This event did not impact the health and safety of the public.

A search of records indicates no previous occurrences of a similar nature.

BOSTON EDISON COMPANY  
800 BOYLSTON STREET  
BOSTON, MASSACHUSETTS 02199

WILLIAM D. HARRINGTON  
SENIOR VICE PRESIDENT  
NUCLEAR

BECO Ltr. #84-059

April 26, 1984

Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Docket Number 50-293  
License DPR-35

Dear Sir:

The attached Licensee Event Report 84-004-00, "Safety Valve Setpoints Below Requirement of Technical Specifications," is hereby submitted in accordance with the requirements of 10CFR50.73.

If there are any questions on this subject, please do not hesitate to contact me.

Respectfully submitted,

*W D Harrington*

W. D. Harrington

PH:caw

Enclosure: LER 84-004-00

cc: Dr. Thomas E. Murley  
Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, PA 19406

Standard BECO LER Distribution

*IE22*  
*11*