

FLORIDA POWER CORPORATION  
CRYSTAL RIVER UNIT 3  
DOCKET NO. 50-302/LICENSE NO. DPR-72  
REQUEST NO. 114, REVISION 0  
LICENSEE EVENT REPORTS

LICENSE DOCUMENT INVOLVED: Technical Specification (Appendix A)

PORION: 1.0 Definitions  
6.5 Review and Audit  
6.6 Reportable Occurrence Action  
6.9 Reporting Requirements  
6.10 Record Retention

DESCRIPTION OF REQUEST:

Delete existing Technical Specification Licensee Event Report requirements as suggested by Generic Letter No. 83-43. In addition to the changes suggested by the Generic Letter, there are several other necessary reporting requirement revisions due to the revised 10 CFR 50.73. Specifically, the statement .... "in lieu of any other report required by Specification 6.9.1".... should be deleted from Specifications 3.3.3.7, 3.7.11.1, 3.7.11.2, 3.7.11.3, and 3.7.11.4. Additionally, the requirement to prepare and submit a REPORTABLE OCCURRENCE in Specification 3.4.8 should be replaced by a Special Report.

Reports on abnormal degradation of the containment structure referenced in Surveillance 4.6.1.6.5 should be reported pursuant to 10 CFR 50.73, not Specification 6.9.1.8.c. In Section 6 where a report (Safety Limit Violation Report or Special Report) must be submitted to the Commission that could also be required by 10 CFR 50.73, include the allowance to use the LER requirements instead of submitting two separate reports.

REASON FOR REQUEST:

A new Section 50.73 to 10 CFR has revised the Licensee Event Report system. The requirement of 50.73 has replaced several existing Technical Specification requirements, necessitating a revision to these Specifications.

By allowing Florida Power Corporation the option to use the LER requirements of 10 CFR 50.73 (B) when submitting a Safety Limit Violation Report or Special Report, unnecessary time and paperwork is eliminated making one report serve the function of two (i.e., where an event requires the submittal of an LER and a Special Report.)

EVALUATION OF REQUEST:

This change is clarifying and made at the request of the Commission, per Generic Letter 83-43, and, therefore, does not affect plant safety.

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## DEFINITIONS

### REPORTABLE EVENT:

1.7 A REPORTABLE EVENT shall be any of those conditions specified in Section 50.73 to 10 CFR Part 50.

### CONTAINMENT INTEGRITY

- 1.8 CONTAINMENT INTEGRITY shall exist when:
- a. All penetrations required to be closed during accident conditions are either:
    1. Capable of being closed by an OPERABLE containment automatic isolation system, or
    2. Closed by manual valves, blind flanges, or deactivated automatic valves secured in their closed positions, except as provided in Table 3.6-1 of Specification 3.6.3.1.
  - b. All equipment hatches are closed and sealed,
  - c. Each airlock is OPERABLE pursuant to Specification 3.6.1.3,
  - d. The containment leakage rates are within the limits of Specification 3.6.1.2, and
  - e. The sealing mechanism associated with each penetration (e.g., welds, bellows or O-rings) is OPERABLE.

### CHANNEL CALIBRATION

1.9 A CHANNEL CALIBRATION shall be the adjustment, as necessary, of the channel output such that it responds with necessary range and accuracy to known values of the parameter which the channel monitors. The CHANNEL CALIBRATION shall encompass the entire channel including the sensor and alarm and/or trip functions, and shall include the CHANNEL FUNCTIONAL TEST. CHANNEL CALIBRATION may be performed by any series of sequential, overlapping or total channel steps such that the entire channel is calibrated.

### CHANNEL CHECK

1.10 A CHANNEL CHECK shall be the qualitative assessment of channel behavior during operation by observation. This determination shall include, where possible, comparison of the channel indication and/or status with other indications and/or status derived from independent instrument channels measuring the same parameter.

## INSTRUMENTATION

### FIRE DETECTION INSTRUMENTATION

#### LIMITING CONDITION FOR OPERATION

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3.3.3.7 As a minimum, the fire detection instrumentation for each fire detection zone shown in Table 3.3-11 shall be OPERABLE.

**APPLICABILITY:** Whenever equipment in that fire detection zone is required to be OPERABLE.

#### **ACTION:**

With one or more of the fire detection instrument(s) shown in Table 3.3-11, inoperable:

- a. Within 1 hour, establish a fire watch patrol to inspect the zone(s) with the inoperable instrument(s) at least once per hour, and
- b. Restore the inoperable instrument(s) to OPERABLE status within 14 days or prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the action taken, the cause of the malfunction and the plans and schedule for restoring the instrument(s) to OPERABLE status.
- c. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

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4.3.3.7.1 Each of the above fire detection instruments shall be demonstrated OPERABLE at least once per 6 months by performance of a CHANNEL FUNCTIONAL TEST.

4.3.3.7.2 The circuitry associated with the detector alarms listed in Table 3.3-11 shall be demonstrated OPERABLE at least once per 6 months for all National Fire Protection Association (NFPA) Code 72D Class B supervised circuits.

4.3.3.7.3 The non-supervised circuits between the local panels and the control room for the detectors listed in Table 3.3-11 shall be demonstrated OPERABLE at least once per 31 days.

## REACTOR COOLANT SYSTEM

### SURVEILLANCE REQUIREMENTS (Continued)

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- b. The complete results of the steam generator tube inservice inspection shall be submitted to the Commission in a Special Report pursuant to Specification 6.9.2 within 12 months following the completion of the inspection. This Special Report shall include:
    - 1. Number and extent of tubes inspected.
    - 2. Location and percent of wall-thickness penetration for each indication of an imperfection.
    - 3. Identification of tubes plugged.
  - c. Results of steam generator tube inspections which fall into Category C-3 shall be reported to the Commission pursuant to 10 CFR 50.72 prior to resumption of plant operation. The written followup of this report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures taken to prevent recurrence pursuant to 10 CFR 50.73.
- 4.4.5.6 The steam generator shall be demonstrated OPERABLE by verifying steam generator level to be within limits at least once per 12 hours.

TABLE 4.4-2

## STEAM GENERATOR TUBE INSPECTION

1ST SAMPLE INSPECTION			2ND SAMPLE INSPECTION		3RD SAMPLE INSPECTION	
Sample Size	Result	Action Required	Result	Action Required	Result	Action Required
A minimum of 5 tubes per S.G.	C-1	None	N/A	N/A	N/A	N/A
	C-2	Plug defective tubes and inspect additional 25 tubes in this S.G.	C-1	None	N/A	N/A
			C-2	Plug defective tubes and inspect additional 45 tubes in this S.G.	C-1	None
			C-3	Perform action for C-3 result of first sample.	C-2	Plug defective tubes.
	C-3	Perform action for C-3 result of first sample.				
	C-3	Inspect all tubes in this S.G., plug defective tubes, and inspect 25 tubes in each other S.G.  Notify NRC pursuant to 10 CFR 50.72.	All other S.G.s are C-1.	None	N/A	N/A
Some S.G.s C-2 but no additional S.G.s are C-3.			Perform action for C-2 result of second sample.	N/A	N/A	
Additional S.G. is C-3.			Inspect all tubes in each S.G. and plug defective tubes. Notify NRC pursuant to 10 CFR 50.72.	N/A	N/A	

$S = 3 \frac{N}{n} \%$  Where **N** is the number of steam generators in the unit and **n** is the number of steam generators inspected during an inspection.



## REACTOR COOLANT SYSTEM

### SPECIFIC ACTIVITY

#### LIMITING CONDITION FOR OPERATION

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3.4.8 The specific activity of the primary coolant shall be limited to:

- a.  $\leq 1.0 \mu\text{Ci/gram DOSE EQUIVALENT I-131}$ , and
- b.  $\leq 100/\bar{E} \mu\text{Ci/gram}$

APPLICABILITY: MODES 1, 2, 3, 4 and 5.

#### ACTION:

MODES 1, 2 and 3\*

- a. With the specific activity of the primary coolant  $> 1.0 \mu\text{Ci/gram DOSE EQUIVALENT I-131}$  but within the allowable limit (below and to the left of the line) shown on Figure 3.4-1, operation may continue for up to 48 hours provided that operation under these circumstances shall not exceed 10% of the unit's total yearly operating time. The provisions of Specification 3.0.4 are not applicable.
- b. With the specific activity of the primary coolant  $> 1.0 \mu\text{Ci/gram DOSE EQUIVALENT I-131}$  for more than 48 hours during one continuous time interval or exceeding the limit line shown on Figure 3.4-1, be in at least HOT STANDBY with  $T_{\text{avg}} < 500^\circ\text{F}$  within 6 hours.
- c. With the specific activity of the primary coolant  $> 100/\bar{E} \mu\text{Ci/gram}$ , be in at least HOT STANDBY with  $T_{\text{avg}} < 500^\circ\text{F}$  within 6 hours.

MODES 1, 2, 3, 4 and 5:

- a. With the specific activity of the primary coolant  $> 1.0 \mu\text{Ci/gram DOSE EQUIVALENT I-131}$  or  $> 100/\bar{E} \mu\text{Ci/gram}$ , perform the sampling and analysis requirements of item 4 a) of Table 4.4-4 until the specific activity of the primary coolant is restored to within its limits. A Special Report shall be prepared and submitted to the Commission pursuant to Specification 6.9.2 within the next 30 days. This report shall contain the results of the specific activity analyses together with the following information:

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\*With  $T_{\text{avg}} \geq 500^\circ\text{F}$



## CONTAINMENT SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

- a. Measuring the elevation difference of 7 dome survey points (1 at the apex; 3 at a radius of  $\approx 29$  feet at azimuths  $90^{\circ}$ ,  $215^{\circ}$  and  $334^{\circ}$ ; and 3 at a radius of  $\approx 49$  feet at azimuths  $90^{\circ}$ ,  $215^{\circ}$  and  $334^{\circ}$ ) and 3 benchmarks (on Ring Girder at azimuths  $90^{\circ}$ ,  $215^{\circ}$  and  $334^{\circ}$ ) along the respective azimuths. These elevation differences shall be compared to the elevation differences established by the Baseline Survey. If the containment is in a normal operation/shutdown mode, the acceptable change in elevation differences will be based on consideration of expected movement and survey accuracy coupled with an acceptable strain level for the radial reinforcement. Changes of a greater magnitude shall require an engineering evaluation. If the containment is in a pressurized mode for a periodic containment integrated leak rate test, the acceptable changes in elevation differences will be similar to that for the initial containment structural integrity test applied to the elevation differences during the periodic containment integrated leak rate test.
- b. Measuring crack widths and plotting crack patterns in the area of the dome 3 feet on either side of azimuths  $195^{\circ}$  from the apex to the Ring Girder. Cracks wider than 0.010 inches will be plotted and cracks wider than 0.040 inches shall require an engineering evaluation. In addition, a general visual inspection of the entire dome surface area shall be performed.

4.6.1.6.5 Reports Any abnormal degradation of the containment structure detected during the above required tests and inspections shall be reported to the Commission pursuant to 10 CFR 50.73. This report shall include a description of the tendon condition, the condition of the concrete (especially at tendon anchorages), the inspection procedures, the tolerances on cracking, and the corrective actions taken.

## PLANT SYSTEMS

### 3/4 7.11 FIRE SUPPRESSION SYSTEMS

#### FIRE SUPPRESSION WATER SYSTEM

#### LIMITING CONDITION FOR OPERATION

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3.7.11.1 The fire suppression water system shall be OPERABLE with:

- a. At least two high-pressure pumps each with a capacity of 2000 gpm, with their discharge aligned to the fire suppression header.
- b. Separate water supplies, each with a minimum contained water volume of 346,000 gallons.
- c. An OPERABLE flow path capable of taking suction from the water supply and transferring the water through distribution piping with OPERABLE sectionalizing control or isolation valves to the yard hydrant curb valves and the front valve ahead of the water flow alarm device on each sprinkler, hose standpipe and spray system riser required to be OPERABLE per Specifications 3.7.11.2 and 3.7.11.4.

APPLICABILITY: At all times.

#### ACTION:

- a. With one pump and/or one water supply inoperable, restore the inoperable equipment to OPERABLE Status within 7 days or prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the plans and procedures to be used to provide for the loss of redundancy in this system. The provisions of Specification 3.0.3 and 3.0.4 are not applicable.
- b. With the fire suppression water system otherwise inoperable:
  1. Establish a backup fire suppression water system within 24 hours, and
  2. Submit a Special Report in accordance with Specification 6.9.2:
    - a. By telephone within 24 hours,
    - b. Confirm by telegraph, mailgram, or facsimile transmission no later than the first working day following the event, and
    - c. In writing within 14 days following the event, outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

## PLANT SYSTEMS

### DELUGE AND SPRINKLER SYSTEMS

#### LIMITING CONDITION FOR OPERATION

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3.7.11.2 The deluge and sprinkler systems shown in Table 3.7-4 shall be OPERABLE.

**APPLICABILITY:** Whenever equipment in the deluge/sprinkler protected areas is required to be OPERABLE.

**ACTION:**

- a. With one or more of the above required deluge and sprinkler systems inoperable, establish a continuous fire watch with backup fire suppression equipment for the unprotected area(s) within 1 hour; restore the system to OPERABLE status within 14 days, or prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.
- b. The provisions of Specification 3.0.3 and 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

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4.7.11.2 Each of the above required deluge and sprinkler systems shall be demonstrated OPERABLE:

- a. At least once per 12 months by cycling each testable valve in the flow path through at least one complete cycle of full travel.
- b. At least once per 18 months:
  1. By performing a system functional test which includes simulated automatic actuation of the system, and;
  2. Verifying that the automatic valves in the flow path actuate to their correct positions.

## PLANT SYSTEMS

### HALON SYSTEM

#### LIMITING CONDITON FOR OPERATION

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3.7.11.3 The Halon system in the Cable Spreading room (Control Complex, Elevation 134'0") shall be OPERABLE with the storage tanks having at least 95% of full charge weight and 90% of full charge pressure.

APPLICABILITY: At all times.

ACTION:

- a. With the above required Halon system inoperable, establish a continuous fire watch with backup fire suppression equipment for the unprotected area within 1 hour; restore the system to OPERABLE status within 14 days or, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.
- b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

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4.7.11.3 The Halon system shall be demonstrated OPERABLE:

- a. At least once per 6 months by verifying each Halon storage tank weight and pressure.
- b. At least once per 18 months by
  1. Verifying the system, including associated ventilation dampers, would actuate automatically to a simulated test signal.
  2. Verifying the OPERABILITY of the manual initiating system.
  3. Performance of a flow test through headers and nozzles to assure no blockage.

## ADMINISTRATIVE CONTROLS

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### QUORUM

- 6.5.1.5 A quorum of the PRC shall consist of the Chairman or his designated alternate and five members including alternates.

### RESPONSIBILITIES

- 6.5.1.6 The Plant Review Committee shall be responsible for:
- a. Review of 1) all procedures and changes thereto as required by Specification 6.8.2, 2) any other proposed procedures or changes thereto as determined by the Nuclear Plant Manager to affect nuclear safety.
  - b. Review of all proposed tests and experiments that affect nuclear safety.
  - c. Review of all proposed changes to the Appendix "A" Technical Specifications.
  - d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
  - e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President, Nuclear Operations and to the Chairman of the Nuclear General Review Committee.
  - f. Review of all REPORTABLE EVENTS.
  - g. Review of facility operations to detect potential nuclear safety hazards.
  - h. Performance of special reviews, investigations, or analyses and reports thereon as requested by the Chairman of the Nuclear General Review Committee.
  - i. Review of the Plant Security Plan and implementing procedures.
  - j. Review of the Emergency Plan and implementing procedures.

## ADMINISTRATIVE CONTROLS

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### REVIEW (Continued)

- g. All REPORTABLE EVENTS.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meetings minutes of the Plant Review Committee.

### AUDITS

6.5.2.9 Audits of facility activities shall be performed under the cognizance of the NGRC. These Audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire facility staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
- e. The Facility Emergency Plan and implementing procedures at least once per 24 months.
- f. The Facility Security Plan and implementing procedures at least once per 24 months.
- g. The Facility Fire Protection Program and implementing procedures at least once per 24 months.
- h. Any other area of facility operation considered appropriate by the NGRC or the Senior Vice President-Engineering and Construction.

### AUTHORITY

6.5.2.10 The NGRC shall report to and advise the Senior Vice President, Engineering and Construction, on those areas of responsibility specified in Sections 6.5.2.8 and 6.5.2.9.



## ADMINISTRATIVE CONTROLS

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### RECORDS

6.5.2.11 Records of NGRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NGRC meeting shall be prepared, approved and forwarded to the Senior Vice President, Engineering and Construction, within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.8 above, shall be prepared, approved and forwarded to the Senior Vice President, Engineering and Construction, within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.9 above, shall be forwarded to the Senior Vice President, Engineering and Construction, and to the management positions responsible for the areas audited within 30 days after completion of the audit.

### 6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the PRC and submitted to the NGRC and the Vice President, Nuclear Operations.



## ADMINISTRATIVE CONTROLS

### 6.7 SAFETY LIMIT VIOLATION

- 6.7.1 The following actions shall be taken in the event a Safety Limit is violated:
- The facility shall be placed in at least HOT STANDBY within one hour.
  - The Safety Limit violation shall be reported to the Commission, the Vice President, Nuclear Operations, and to the NGRC within 24 hours.
  - A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PRC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures and (3) corrective action taken to prevent recurrence.
  - The Safety Limit Violation Report shall be submitted to the Commission, the NGRC and the Vice President, Nuclear Operations within 14 days of the violation. A separate Licensee Event Report need not be submitted if the Safety Limit Violation Report meets the requirements of 10 CFR 50.73 (B) in addition to the requirements above.

### 6.8 PROCEDURES

- 6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:
- The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November, 1972.
  - Refueling operations.
  - Surveillance and test activities of safety related equipment.
  - Security Plan implementation.
  - Emergency Plan implementation.
  - Fire Protection Program implementation.
  - Systems Integrity Program implementation.
  - Iodine Monitoring Program implementation.
- 6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed and approved prior to implementation as follows:
- The Emergency Plan, Security Plan, Fire Protection Plan and implementing procedures, Administrative Instructions and those test procedures associated with plant modifications shall be reviewed and approved by the PRC and the Nuclear Plant Manager prior to implementation.

## ADMINISTRATIVE CONTROLS

- 6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:
- The intent of the original procedure is not altered.
  - The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License.
  - The change is documented and subsequently reviewed and approved within 14 days of implementation, in accordance with the requirements of Specification 6.8.2.

## 6.9 REPORTING REQUIREMENTS

### ROUTINE REPORTS

- 6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Director of the Regional Office of Inspection and Enforcement unless otherwise noted.

### STARTUP REPORTS

- 6.9.1.1 A summary report of plant startup and power escalation testing will be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.
- 6.9.1.2 The startup report shall address each of the tests identified in the FSAR and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details requested in license conditions based on other commitments shall be included in this report.
- 6.9.1.3 Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events, (i.e., initial criticality, completion of startup test program, and the resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

## ADMINISTRATIVE CONTROLS

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### MONTHLY OPERATING REPORT

- 6.9.1.6 Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Director, Office of Management Information and Program Control, U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the Regional Office, submitted no later than the 15th of each month following the calendar month covered by the report.

DELETED

## ADMINISTRATIVE CONTROLS

### SPECIAL REPORTS

- 6.9.2 Special reports shall be submitted to the Director of the Office of Inspection and Enforcement, Region II, within the time period specified for each report. These reports shall be submitted covering the activities identified below. A separate Licensee Event Report, when required by 10 CFR 50.73 (A), need not be submitted if the Special Report meets the requirements of 10 CFR 50.73 (B) in addition to the requirements of the applicable reference Specification.
- a. ECCS Actuation, Specification 3.5.2 and 3.5.3.
  - b. Inoperable Seismic Monitoring Instrumentation, Specification 3.3.3.3.
  - c. Inoperable Meteorological Monitoring Instrumentation, Specification 3.3.3.4.
  - d. Seismic event analysis, Specification 4.3.3.3.2.
  - e. Inoperable Fire Detection Monitoring Instrumentation, Specification 3.3.3.7.
  - f. Inoperable Fire Suppression System, Specification 3.7.11.1., 3.7.11.2, and 3.7.11.3.
  - g. Specific Activity, Specification 3.4.8.
  - h. Results of Steam Generator Tube Inspection, Specification 4.4.5.5.b.

## ADMINISTRATIVE CONTROLS

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### 6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time intervals at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. All REPORTABLE EVENTS submitted to the Commission.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of reactor tests and experiments.
- f. Records of changes made to Operating Procedures.
- g. Records of radioactive shipments.
- h. Records of sealed source and fission detector leak tests and results.
- i. Records of annual physical inventory of all sealed source material of record.

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

- a. Records and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of facility radiation and contamination surveys.
- d. Records of radiation exposure for all individuals entering radiation control areas.

## NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION

**Docket No:** 50-302

**Facility:** Crystal River Unit 3

**Licensee:** Florida Power Corporation

**Date of Application:**

### **Request**

Florida Power Corporation requests issuance of an amendment to the Crystal River Unit 3 Technical Specifications deleting Licensee Event Report requirements.

### **Significant Hazards Consideration Determination:**

- ( x )      Amendment involves no significant hazards considerations.  
(   )      Amendment involves significant hazards considerations.

### **Basis for Determination:**

On January 1, 1984, 10 CFR 50.73 revised the Licensee Event Report system, replacing existing Technical Specifications. This amendment is requested to make the license conform to changes in regulations.

Therefore, this amendment is considered not likely to involve significant hazards considerations.

### **Requested Implementation Date:**

Florida Power Corporation does not request an implementation date.