

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2										DOCKET NUMBER (2) 0 5 0 0 0 3 6 1				PAGE (3) 1 OF 0 1		
TITLE (4) HIGH STEAM GENERATOR LEVEL TRIP																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
0 3	2 6	8 4	8 4	0 2 0	0 0	0 4	2 4	8 4					0 5 0 0 0 0 0 0			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)														
2		20.402(b)				20.405(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)		73.71(b)				
POWER LEVEL (10)		0 0 2				20.405(a)(1)(i)				50.36(c)(1)		73.71(c)				
		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)				
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)						
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)						
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME J. G. HAYNES, STATION MANAGER										TELEPHONE NUMBER 7 1 4 4 9 2 - 7 7 0 0						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC						
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO				

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 0520 on March 26, 1984, Unit 2 was returning to service following a reactor trip which occurred two days earlier. The Main Feedwater (MFW) System (EIIS System Code SJ) remained in service after the trip since decay heat was adequate to support a main feedwater pump turbine. As a result, the steam generator levels were being maintained by manual operation of the Feedwater Control System (FWCS) (EIIS System Code JB). At 0633, with the Unit in Mode 2 at 2 percent power, a reactor trip occurred on high steam generator level. The trip occurred as a result of overfeeding the steam generators during manual operation of the Feedwater Control System (FWCS) (EIIS System Code JB). The Plant Protection System (PPS) (EIIS System Code JC) responded normally to stabilize plant conditions during this event. No system or component malfunctioned during this event.

Manual steam generator level control is difficult at low power due to the "shrink" and "swell" responses of steam generator levels. Design changes to optimize steam generator level control at all power levels are under consideration.

There are no reasonable or credible alternatives under which this event would have been more severe.

*Southern California Edison Company*



SAN ONOFRE NUCLEAR GENERATING STATION

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SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES  
STATION MANAGER

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April 24, 1984

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Subject: Docket No. 50-361  
30-Day Report  
Licensee Event Report No. 84-020  
San Onofre Nuclear Generating Station, Unit 2

Pursuant to 10 CFR 50.73(a)(2)(iv), this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving actuation of the Reactor Protection System. The health and safety of plant personnel or the public were not affected by this event.

If you require any additional information, please so advise.

Sincerely,

Enclosure: LER 84-020

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)  
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, NRC Region V)

Institute of Nuclear Power Operations (INPO)

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