

File

To:

James P. O'Reilly
Directorate of Regulatory Operations
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

From:

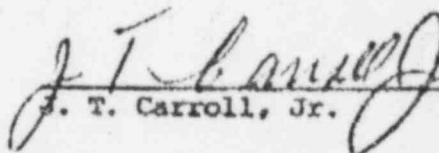
Jersey Central Power & Light Company
Oyster Creek Nuclear Generating Station
Docket #50-219
Forked River, New Jersey 08731

Subject:

Abnormal Occurrence Report No. 50-219/74/52

The following is a preliminary report being submitted
in compliance with the Technical Specifications,
paragraph 6.6.2.

Preliminary Approval:

 10/15/74
J. T. Carroll, Jr. Date

cc: Mr. A Giambusso

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Initial Written
Report Date:

10/15/74

Time of
Occurrence:

0100

OYSTER CREEK NUCLEAR GENERATING STATION
FORKED RIVER, NEW JERSEY 08731

Abnormal Occurrence
Report No. 50-219/74/ 52

IDENTIFICATION
OF OCCURRENCE:

Violation of the Technical Specifications, paragraph 2.3.7,
Main Steam Line Low Pressure Switch RE23A was found to trip
at a pressure less than the minimum required value of 860 psig.

This event is considered to be an abnormal occurrence as de-
fined in the Technical Specifications, paragraph 1.15A.

CONDITIONS PRIOR
TO OCCURRENCE:

<input type="checkbox"/> Steady State Power	<input type="checkbox"/> Routine Shutdown
<input type="checkbox"/> Hot Standby	<input type="checkbox"/> Operation
<input type="checkbox"/> Cold Shutdown	<input type="checkbox"/> Load Changes During
<input type="checkbox"/> Refueling Shutdown	<input type="checkbox"/> Routine Power Operation
<input type="checkbox"/> Routine Startup	<input checked="" type="checkbox"/> Other (Specify)
<input type="checkbox"/> Operation	

The plant was shutdown with reactor coolant at $<212^{\circ}\text{F}$

DESCRIPTION
OF OCCURRENCE:

On Saturday, October 12, 1974, at 0100, while performing a rou-
tine surveillance test on the four Main Steam Line Low Pressure
Switches, it was discovered that switch RE23A tripped at 855 psig.
This value is below the minimum required trip point of 860 psig,
which is derived by adding to the Technical Specification limit
of 850 psig a 10 psig head correction factor.

The "as found" and "as left" switch settings were:

	<u>"As Found" Settings</u>	<u>"As Left" Settings</u>
RE23A	655 psig	860 psig
RE23B	870 psig	860 psig
RE23C	860 psig	860 psig
RE23D	860 psig	860 psig

APPARENT CAUSE
OF OCCURRENCE:

<input type="checkbox"/> Design	<input type="checkbox"/> Procedure
<input type="checkbox"/> Manufacture	<input type="checkbox"/> Unusual Service Condition
<input type="checkbox"/> Installation	<input type="checkbox"/> Inc. Environmental
<input type="checkbox"/> Construction	<input type="checkbox"/> Component Failure
<input type="checkbox"/> Operator	<input checked="" type="checkbox"/> Other (Specify)

The cause of this occurrence is switch repeatability which is a recognized problem.

ANALYSIS OF
OCCURRENCE:

Four pressure switches in the reactor protection system (RPS) are provided to isolate the reactor in the event of low main steam line pressure. Two of these switches (RE23A and RE23C) are in RPS Channel 1 and the other two switches (RE23B and RE23D) are in RPS Channel 2. Low pressure signals from two pressure switches, one in each RPS channel, are required to effect main steam isolation valve closure. A review of the "as-found" switch settings indicates that reactor isolation would have occurred at the minimum required pressure had a main steam line low pressure condition existed. Consequently, this event is considered to have no safety significance.

**CORRECTIVE
ACTION:**

Steam line pressure variations during daily stop valve testing make it impractical to include in the switch setpoint the normal trip point variations to provide a sufficient margin above the Technical Specification limit.

The plant has received from General Electric Company information to enable us to submit a Technical Specification Change Request which will encompass changing the present 850 psig setpoint of the Main Steam Line Low Pressure Switches to a lower setpoint.

As this proposal was received on September 27, 1974, it has not yet been fully evaluated by the plant staff. A Technical Specification Change Request may be submitted pending the results of the plant staff evaluation.

Previous abnormal occurrence reports involving these switches are:

1. Letter to Mr. A. Giambusso from Mr. D. A. Ross, dated December 24, 1973.
2. Abnormal Occurrence Report No. 74-1.
3. Abnormal Occurrence Report No. 74-9.
4. Abnormal Occurrence Report No. 74-10
5. Abnormal Occurrence Report No. 74-12.
6. Abnormal Occurrence Report No. 74-22

7. Abnormal Occurrence Report No. 74-35.
8. Abnormal Occurrence Report No. 74-37.
9. Abnormal Occurrence Report No. 74-41.
10. Abnormal Occurrence Report No. 74-42.
11. Abnormal Occurrence Report No. 74-43.
12. Abnormal Occurrence Report No. 74-49.
13. Abnormal Occurrence Report No. 74-51.

FAILURE DATA:

Manufacturer data pertinent to these switches are as follows:

Meletron Corporation (subsidiary of Barksdale)
Los Angeles, California
Pressure Actuated Switch
Model No. 372
Catalog No. 372-6SS49A-293
Range 20-1400 psig
Proof psi 1750 G

Prepared by: J. E. Manning

Date: 10/15/74