

Jersey Central Power & Light Company



MADISON AVENUE AT PUNCH BOWL ROAD • MORRISTOWN, N. J. 07960 • 201-539-6111

MEMBER OF THE

General



Public Utilities Corporation

October 21, 1974



Mr. A. Giambusso
Deputy Director for Reactor Projects
Directorate of Licensing
United States Atomic Energy Commission
Washington, D. C. 20545

Dear Mr. Giambusso:

Subject: Oyster Creek Station
Docket No. 50-219
Abnormal Occurrence Report No. 50-219/74-52

The purpose of this letter is to forward to you the attached Abnormal Occurrence Report in compliance with paragraph 6.6.2.a of the Technical Specifications.

Enclosed are forty copies of this submittal.

Very truly yours,

Donald A. Ross
Manager, Nuclear Generating Stations

pk
Enclosures

cc: Mr. J. P. O'Reilly, Director
Directorate of Regulatory Operations, Region 1

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SYSTEM

OYSTER CREEK NUCLEAR GENERATING STATION
FORKED RIVER, NEW JERSEY 08731

Abnormal Occurrence
Report No. 50-219/74-52

Report Date

October 21, 1974

Occurrence Date

October 12, 1974

Identification of Occurrence

Violation of the Technical Specifications, paragraph 2.3.7, Main Steam Line Low Pressure Switch RE23A was found to trip at a pressure less than the minimum required value of 860 psig. This event is considered to be an abnormal occurrence as defined in the Technical Specifications, paragraph 1.15A.

Conditions Prior to Occurrence

The plant was shut down with reactor coolant at a temperature less than 212°F.

Description of Occurrence

On Saturday, October 12, 1974, at 0100, while performing a routine surveillance test on the four Main Steam Line Low Pressure Switches, it was discovered that switch RE23A tripped at 855 psig. This value is below the minimum required trip point of 860 psig, which is derived by adding to the Technical Specification limit of 850 psig a 10 psig head correction factor.

The "as found" and "as left" switch settings were:

	<u>"As Found" Settings</u>	<u>"As Left" Settings</u>
RE23A	855 psig	860 psig
RE23B	870 psig	860 psig
RE23C	860 psig	860 psig
RE23D	860 psig	860 psig

Apparent Cause of Occurrence

The cause of this occurrence is the recognized problem of switch repeatability.

Analysis of Occurrence

Four pressure switches in the reactor protection system (RPS) are provided to isolate the reactor in the event of low main steam line pressure. Two of these switches (RE23A and RE23C) are in RPS Channel 1, and the other two switches (RE23B and RE23D) are in RPS Channel 2. Low pressure signals from two pressure switches, one in each RPS channel, are required to effect main steam isolation valve closure. A review of the "as found" switch settings indicates that reactor isolation would have occurred at the minimum required pressure had a main steam line low pressure condition existed. Consequently, this event is considered to have no safety significance.

Corrective Action

Steam line pressure variations during daily stop valve testing make it impractical to include in the switch setpoint the normal trip point variations to provide a sufficient margin above the Technical Specification limit.

The General Electric Company has furnished information on the feasibility of reducing the 850 psig setpoint to a lower value. This information is being evaluated by the Oyster Creek Nuclear Station staff, and a Technical Specification Change Request may be submitted as a result of this evaluation.

Failure Data

Manufacturer data pertinent to these switches are as follows:

Meletron Corporation (subsidiary of Barksdale)
Los Angeles, California
Pressure Actuated Switch
Model 372
Catalog No. 372-6SS49A-293
Range 20-1400 psig
Proof Psi 1750 G

Previous abnormal occurrence reports involving these switches are:

1. Letter to Mr. A. Giambusso from D. A. Ross, dated December 24, 1973.
2. Abnormal Occurrence Report No. 50-219/74-1.
3. Abnormal Occurrence Report No. 50-219/74-9.
4. Abnormal Occurrence Report No. 50-219/74-10.
5. Abnormal Occurrence Report No. 50-219/74-12.
6. Abnormal Occurrence Report No. 50-219/74-22.
7. Abnormal Occurrence Report No. 50-219/74-35.
8. Abnormal Occurrence Report No. 50-219/74-37.
9. Abnormal Occurrence Report No. 50-219/74-41.
10. Abnormal Occurrence Report No. 50-219/74-42.
11. Abnormal Occurrence Report No. 50-219/74-43.
12. Abnormal Occurrence Report No. 50-219/74-49.
13. Abnormal Occurrence Report No. 50-219/74-51.