

SNUPPS

Standardized Nuclear Unit
Power Plant System

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Nicholas A. Petrick
Executive Director

April 24, 1984

SLNRC 84- 0074 FILE: 0278
SUBJ: NUREG-0737 II.B.3, Post-
Accident Sampling Capability

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Docket Nos. STN 50-482 and STN 50-483

- References:
1. SLNRC 83-0007, dated February 4, 1983: Same Subject
 2. NRC Letter (B. Youngblood) to Kansas Gas and Electric Company (G. Koester), dated December 19, 1983: Issuance of Supplement No. 4 to the Wolf Creek Generating Station, Unit 1 Safety Evaluation Report
 3. SLNRC 84-0015, dated February 1, 1984: SNUPPS License Conditions

Dear Mr. Denton:

Reference 1 provided information regarding the post-accident sampling capability of the SNUPPS design in response to License Conditions 20 and B.17 for Callaway Plant Unit No. 1 and Wolf Creek Generating Station Unit No. 1, respectively. The NRC evaluation of Reference 1, which was provided in Reference 2, stated that the SNUPPS post-accident sampling system meets the criteria of Item II.B.3 of NUREG-0737 with the exception of two criteria which have not been fully resolved:

Criterion (6) Provide a core damage estimate procedure to include radionuclide concentrations and other physical parameters as indicators of core damage.

Criterion (9) Provide information demonstrating applicability of procedures and instrumentation in the post-accident water chemistry and radiation environment, and retraining of operators on a semi-annual basis.

In addition, the Reference 2 evaluation required that, prior to exceeding 5% power operation the Post-Accident Sampling System shall be installed and operational and the applicant: (1) shall submit for NRC approval a core damage assessment procedure which incorporates, as a

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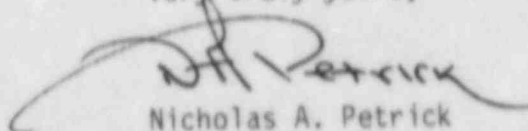
minimum, hydrogen levels, reactor coolant system pressure, core exit thermocouple temperatures and containment radiation levels in addition to radionuclide data; (2) shall demonstrate applicability of procedures and instrumentation in the post-accident water chemistry and radiation environment, and retraining of operators on semi-annual basis.

In response to the remaining NUREG-0737 II.B.3 issues identified above, the following information is provided:

- A. At both Callaway Plant and Wolf Creek Generating Station, the post-accident sampling system will be completely installed and operational prior to initial fuel loading.
- B. Enclosure 1 is a core damage assessment procedure for the Callaway Plant which is responsive to Criterion (6) above. A similar procedure for the Wolf Creek Generating Station will be submitted for NRC review on or before September 1, 1984.
- C. Enclosure 2 provides additional information which demonstrates the applicability of the post-accident sampling system instrumentation and procedures in the post-accident water chemistry and radiation environment.
- D. Retraining of post-accident sampling system operating personnel on a semi-annual basis will be conducted as described in Reference 3.

Based on the above information which addresses each of the NRC staff concerns identified in Reference 2, it is requested that the license conditions regarding post-accident sampling capability be deleted for the SNUPPS plants.

Very truly yours,



Nicholas A. Petrick

MHF/nld10a1&2
Attachment

cc: G. L. Koester
D. T. McPhee
D. F. Schnell
J. Neilser/B. Little
W. Schum/A. Smith
B. Forney
E. Johnson

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