

LICENSEE EVENT REPORT (LER)

APPROVED OMB NO. 3160-0104
EXPIRES 6/3/85FACILITY NAME (1)
Peach Bottom Atomic Power Station - Unit 2

DOCKET NUMBER (2)

05000277

PAGE (3)

1 OF 03

TITLE (4)

Spurious Actuation of the Primary Containment Isolation System

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)	
03	23	84	84	005	00	04	19	84			050000	
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 50. (Check one or more of the following) (11)									
POWER LEVEL (10)			20.402(a)			20.406(a)			<input checked="" type="checkbox"/> 50.73(a)(2)(iv)			73.71(b)
			20.406(a)(1)(ii)			60.73(a)(2)(v)						73.71(a)
			20.406(a)(1)(iii)			60.73(a)(2)(vi)						OTHER (Specify in Abstract below and in Text, NRC Form 366A)
			20.406(a)(1)(iv)			60.73(a)(2)(vii)(A)						
			20.406(a)(1)(v)			60.73(a)(2)(viii)(B)						
			20.406(a)(1)(vi)			60.73(a)(2)(ix)						

LICENSEE CONTACT FOR THIS LER (12)

NAME
B. L. Clark, Senior Engineer - Special Projects

TELEPHONE NUMBER

AREA CODE

215841150117

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
X	JM	RLYG	080	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Abstract 2-84-05

On March 23, 1984, at 3:47 p.m., with Unit 2 at 95% power level, a partial actuation of the Primary Containment Isolation System (PCIS) occurred. Investigation revealed that the actuation was the result of a loss of power to a portion of the PCIS logic. The PCIS is designed as a fail-safe system and operated as such by isolating the inboard isolation valves in Group II-D of the PCIS. The loss of power was due to a blown fuse which resulted from a shorted coil in Relay 16A-K56 (General Electric, Model No. CR120A04002AA). The relay was replaced, verified as operable, and the PCIS was returned to service.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/86

FACILITY NAME (1) Peach Bottom Atomic Power Station Unit 2	DOCKET NUMBER (2) 0 8 0 0 0 2 7 7 8 4 - 0 0 5 - 0 0 0 2 OF 0 3	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		

TEXT (If more space is required, use additional NRC Form 366A (17))

Description of the Event:

On March 23, 1984, at 3:47 p.m., with Unit 2 at 95% power level, the "Group II/III inboard isolation relays not reset" alarm annunciated in the control room, indicating a Group II or Group III Primary Containment Isolation System (PCIS) initiation. Likewise, at the same time, the valve position indications in the control room for the Drywell Floor Drain Sump Pump Discharge Valve, the Drywell Equipment Drain Sump Pump Discharge Valve, and the "A Instrument Nitrogen Supply to the Drywell" Valve were lost.

Investigation revealed that a partial actuation of the Primary Containment Isolation System (an Engineered Safety Feature) occurred in that the inboard isolation valves in Group II-D of the PCIS isolated along with the "A Instrument Nitrogen Supply to the Drywell" Isolation Valve. Further investigation revealed that the actuation was due to a blown fuse which resulted in loss of power to that portion of the PCIS logic which controls the valves that isolated.

Consequences of the Event:

The Primary Containment Isolation System is designed as a fail-safe type of system. Loss of electrical power to the PCIS logic in panel 9-41 caused that portion of the PCIS system to operate as designed by isolating the valves in Group II-D.

Cause of the Event:

The blown fuse and resulting loss of power to the PCIS logic was a result of a shorted coil in relay 16A-K56 (General Electric Type CR120A04002AA). This relay controls the Main Steam Line Drain inboard isolation valve, MO-2-2-74, which was closed at the time of the event and remained in the isolated (safe) condition throughout the event.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104
EXPIRES 8/31/85

FACILITY NAME (1) Peach Bottom Atomic Power Station Unit 2	DOCKET NUMBER (2) 0500027784	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
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TEXT (if more space is required, use additional NRC Form 366A (17))

Corrective Actions:

Upon identification of the shorted coil, relay 16A-K56 was removed from service. Within minutes, the blown fuse was replaced restoring the electrical power to the affected portion of the PCIS logic, except for relay 16A-K56, thereby allowing the operators to return the inboard isolation valves in Group II-D along with the "A" Instrument Nitrogen Supply valve to service. Relay 16A-K56 was replaced, verified as operable and returned to service within 30 minutes.

The Electrical Engineering Division is being requested to evaluate the General Electric Type CR120A relays in the Primary Containment Isolation System with regard to the energized service life to determine whether or not replacement of the relays is necessary.

Previous Similar Occurrences:

LER: 2-84-02

PHILADELPHIA ELECTRIC COMPANY

2301 MARKET STREET

P.O. BOX 8699

PHILADELPHIA, PA. 19101

(215) 841-4000

April 19, 1984

Docket No. 50-277

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Washington, DC 20555

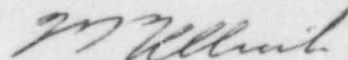
SUBJECT: Licensee Event Report

This LER deals with the spurious actuation of a portion of the Primary Containment Isolation System (PCIS) on Unit 2.

Reference:	Docket No. 50-277
Report Number:	2-84-05
Revision Number:	00
Event Date:	March 23, 1984
Report Date:	April 19, 1984
Facility:	Peach Bottom Atomic Power Station RD #1, Box 208, Delta, PA 17314

This LER is submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(IV).

Very truly yours,



W. T. Ullrich
Superintendent
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator
Region I, USNRC

Mr. A. R. Blough
Site Inspector

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