

Initial Telephone  
Report Date:

Date of  
Occurrence: October 12, 1976

Initial Written  
Report Date:

Time of  
Occurrence:

OYSTER CREEK NUCLEAR GENERATING STATION  
FORKED RIVER, NEW JERSEY 08731

Reportable Occurrence  
Report #50-219-76-26-1P

IDENTIFICATION  
OF OCCURRENCE:

Violation of the Technical Specifications, paragraph 3.6.B.2, in that an unplanned radioactive release to the discharge canal was not monitored. This event is considered to be a prompt notification with written followup reportable occurrence as defined in the Technical Specifications 6.9.2.a.(2).

CONDITIONS PRIOR  
TO OCCURRENCE:

☒ Steady State Power  
☐ Hot Standby  
☐ Cold Shutdown  
☐ Cooling Shutdown  
☐ Routine Startup  
Operation

☐ Routine Shutdown Operation  
☐ Load Changes During Routine  
Power Operation  
☐ Other (Specify)  
\_\_\_\_\_

POWER:

Core - 1841.8 MWt

FLOW:

Elec - 632 MWe

Recirc -  $57.9 \times 10^6$  lbm/hr

Feedwater -  $6.86 \times 10^6$  lbm/hr

STACK GAS:

14,900 uci/sec,

DESCRIPTION  
OF OCCURRENCE:

During the regeneration of a condensate demineralizer on October 12, 1976 it was discovered that the relief valve on the regeneration hot water heater was not seated and was passing condensate transfer system water to the 1-5 sump which is automatically pumped to the environs. This condition existed since the replacement of the relief valve on August 5, 1976 thus resulting in an estimated radioactive release of 8.2 millicuries of activity in 217,000 gallons of condensate water since August 5, 1976.

APPARENT CAUSE  
OF OCCURRENCE:

☒ Design  
☐ Manufacture  
☐ Installation/  
Construction  
☐ Operator

☐ Procedure  
☐ Unusual Service Condition  
☐ Inc. Environmental  
Component Failure  
☐ Other (Specify)  
\_\_\_\_\_

The primary cause of this occurrence was a design error that resulted in the routing of the relief valve discharge to the environs via the 1-5 sump instead of to the radwaste facility via the high conductivity tank. Also instrumental in the cause of the event was the replacement of the original relief valve which did not meet the applicable ASME Code but did function properly and did not leak. The new valve which was installed on August 5, 1976 meets the applicable ASME Code.

ANALYSIS OF  
OCCURRENCE:

The daily 1-5 sump pump running times have shown the valve failure to be a progressive one. The period of October 1 - October 13, was the period of highest release having an average release rate of 6 gpm. With the dilution flow of 460,000 gpm an average concentration of  $1 \times 10^{-10}$  uci/ml would result. This is well below the 10 CFR 20 limit ( $1 \times 10^{-7}$ ) for unidentified mixtures.

CORRECTIVE ACTION:

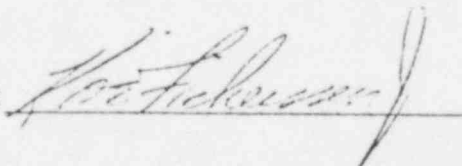
Upon discovery of the leaking relief valve and determination that the discharge was to the 1-5 sump the supply water isolation valve from the condensate transfer system was isolated. Additionally a job order has been initiated to reroute the relief valve discharge to the High Conductivity Tank.

FAILURE DATA:

System Design  
Manufacturer: Burns and Rowe / General Electric Co.

Valve:  
Manufacturer: Kunkle  
Model: 137 Relief Valve  
Rating: 500,000 BTU/HR  
Pressure: 100 psi  
Inlet: 1.5 inches  
Outlet: 2 inches

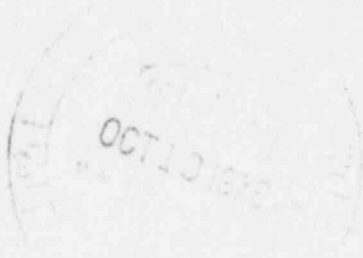
PREPARED BY:



October 13, 1976

TO:

James P. O'Reilly  
Directorate of Regulatory Operations  
Region I  
631 Park Avenue  
King of Prussia, Pennsylvania 19406



FROM:

Jersey Central Power & Light Company  
Oyster Creek Nuclear Generating Station  
Docket No. 50-219  
Forked River, New Jersey 08731

SUBJECT:

Reportable Occurrence Report #50-219-76-26-1P

The following is a preliminary report being  
submitted in compliance with the Technical  
Specifications, paragraph 6.9.2.a.

COPY SENT REGION I

Preliminary Approval.

J. T. Carroll, Jr.  
J. T. Carroll, Jr.  
Station Superintendent

Oct. 13, 1976.

cc. Mr. Roger Boyd