

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Browns Ferry - Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 2 9 6	PAGE (3) 1 OF 0 2
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TITLE (4)
Linear Indication in the Reactor Water Cleanup System

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0	4	0	2	8	4	8	4	0	0	5	0
0	4	0	2	8	4	0	0	5	0	0	0

OPERATING MODE (9) N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)									
POWER LEVEL (10) 0 0 0	20.402(b)	20.406(c)	50.73(a)(2)(iv)	73.71(b)							
	20.406(a)(1)(i)	40.36(c)(1)	50.73(a)(2)(v)	73.71(c)							
	20.406(a)(1)(ii)	40.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)							
	20.406(a)(1)(iii)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(A)								
	20.406(a)(1)(iv)	50.73(a)(2)(iii)	50.73(a)(2)(viii)(B)								
	20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)								

LICENSEE CONTACT FOR THIS LER (12)									
NAME C. J. Rozear								TELEPHONE NUMBER	
								AREA CODE	
								2 1 5	7 1 2 1 9 1 0 7 1 8 1 8

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	C	E	P	S	F	X	9	9	9	N

SUPPLEMENTAL REPORT EXPECTED (14)						EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)									
X NO									

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During performance of the induction heat stress improvement program a linear root indication in the heat affected zone (1/2-inch long by 0.09 inch deep) was found on a six-inch pipe weld in the reactor water cleanup system. The cause of the indication can not be positively determined at this time.

The weld has been evaluated and the results indicated that if the flaw were intergranular stress corrosion cracking, no growth would occur because of the compressive stress field at the crack tip. This weld will be left as is and no repair performed. This action was discussed with the Nuclear Regulatory Commission, Office of Nuclear Reactor Regulation in a telecon on April 9, 1984 and they concurred with TVA's recommendation to leave as is. The weld will be inspected during the next refueling outage and disposition of the weld will be determined after evaluation of the inspection data.

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PDR ADOCK 05000296
S PDR

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO 3150-0104

EXPIRES 8/31/85

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Unit 3 was in cold shutdown for a refueling outage, unit 1 was at 94 percent power, and unit 2 was at 62 percent power. Only unit 3 was affected by this event.

On April 4, 1984, during performance of the induction heat stress improvement program a linear root indication in the heat affected zone (1/2-inch long by 0.09 inch deep) was found in a six-inch pipe weld (appropriate wall thickness 0.43 inch) DSRWC-3-4 on the reactor water cleanup system (CE). The cause of this indication can not be positively determined at this time. The weld has been evaluated and the results indicate no safety problem in that if the flaw were intergranular stress corrosion cracking, no growth would occur because of the compressive stress field at the crack tip. This weld will be left as is and no repair performed. This action was discussed with the Nuclear Regulatory Commission, Office of Nuclear Reactor Regulation, in a telecon on April 9, 1984 and they concurred with TVA's recommendation to leave as is. This weld will be inspected on the next refueling outage and disposition of the weld will be determined after evaluation of the inspection data.

Responsible Plant Section

N/A

Previous Similar Events

BFRO-50-259/83023
BFRO-50-259/83049
BFRO-50-260/82040
BFRO-50-296/83059

TENNESSEE VALLEY AUTHORITY
Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, Alabama 35602

April 20, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

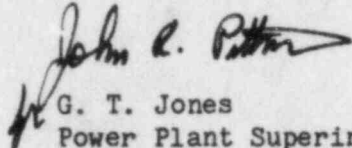
Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 3 - DOCKET
NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE OCCURRENCE
REPORT BFR0-50-296/84005

The enclosed report provides details concerning linear indication in the
reactor water cleanup system. This report is submitted in accordance with
10 CFR 50.73 (a)(2)(ii).

Very truly yours,

TENNESSEE VALLEY AUTHORITY



G. T. Jones
Power Plant Superintendent
Browns Ferry Nuclear Plant

Enclosure

cc (Enclosure):
Regional Administrator
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
101 Marietta Street, Suite 2900
Atlanta, GA 30303

NRC Inspector, Browns Ferry Nuclear Plant

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