SNUPPS

Standardized Nuclear Unit Power Plant System

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Nicholas A. Petrick
Executive Director

April 23, 1984

SLNRC 84-72 FILE: 0543
SUBJ: SNUPPS Technical Specifications Use of RHR Suction Relief Valves for Cold Overpressurization Protection

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Docket Nos. STN 50-482 and STN 50-483

Ref: 1. SLNRC 84-29, 2/10/84: SNUPPS Technical Specifications
2. ULNRC 792, 4/9/84: Callaway Technical Specifications
3. ULNRC 787, 4/5/84: Callaway Plant Technical Specifications - Appeals

Dear Mr. Denton:

The referenced letters forwarded changes requested by the SNUPPS (Callaway and Wolf Creek) utilities to the Standard Technical Specifications that would allow use of the RHR Suction Relief Valves for cold overpressure protection in Modes 4, 5, and 6. Since these submittals, the SNUPPS utilities and the NRC Reactor Systems Branch have discussed this request on two occasions, a telecon on April 10, 1984 and a meeting on April 13, 1984. These discussions resulted in the following agreements:

1) Technical Specifications: 3.4.9.3 and its associated surveillances, 3.8.1.2, 3.8.2.2, 3.8.3.2, Bases 3/4.1.2, and B3/4.4.9 will be accepted as submitted on a temporary basis until the startup following the first refueling outage (except that the basis will be revised to provide more information on cold overpressure protection).

2) The SNUPPS utilities will incorporate administrative controls to independently verify proper RHR suction valve alignment prior to exceeding a primary pressure of 600psig. The controls will be incorporated into the Plant Operating Procedures.

3) The SNUPPS utilities will evaluate permanent measures to comply with Branch Technical Position 5-1 including but not limited to the following:

a) evaluating the use of 2/3 pressure logic signals on each RHR suction valve to provide redundancy,

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b. evaluating the use of a second interlock such that if both valves in a train are either open or closed, a high pressure signal to one valve will not cause isolation,

c. and evaluating the use of administrative controls for ensuring proper valve alignment based on plant conditions.

The evaluation will be discussed with and submitted formally to the NRC prior to January 1, 1985 and the resulting decision implemented prior to the startup following the first refueling outage.

Very truly yours,

Nicholas A. Petrick

JHR/dck/2b1

Attachment

cc: J. Neisler/B. Little, USNRC/CAL
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