

EXPIRES 5/31/96

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HOURS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON DC 20503.

FACILITY NAME (1) CRYSTAL RIVER UNIT 3 (CR-3)										DOCKET NUMBER (2) 0 5 0 0 0 3 0 2				PAGE (3) 1 OF 0 5										
TITLE (4) Inattention to Detail Causes Delay in Performance of Surveillance Resulting in Violation of Improved Technical Specifications																								
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)														
MONTH	DAY	YEAR	YEAR	SEQUENTIA NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES N/A				DOCKET NUMBER(S) 0 5 0 0 0											
0	4	2	1	9	5	9	5	0	0	7	0	0	0	5	1	9	9	5	N/A	0	5	0	0	0
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (CHECK ONE OR MORE OF THE FOLLOWING) (11)																						
1		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)										
POWER LEVEL (10)		20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)										
1		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 386A)										
		20.405(a)(1)(iii)				X 50.73(a)(2)(i)				50.73(a)(2)(viii)(A)														
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)														
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)														
LICENSEE CONTACT FOR THIS LER (12)																								
NAME J. A. Frijouf, Nuclear Regulatory Specialist										TELEPHONE NUMBER AREA CODE 9 0 4 5 6 3 - 4 7 5 4														
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE IN THIS REPORT (13)																								
CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER	REPORTABLE TO NPRDS														
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR									
YES (If yes, complete EXPECTED SUBMISSION DATE)												X NO												

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On April 21, 1995, Florida Power Corporation's (FPC) Crystal River Unit 3 (CR-3) was in MODE ONE (POWER OPERATION), operating at 100% reactor power and generating 860 megawatts. At that time, it was determined that CR-3 had operated outside its Improved Technical Specifications (ITS) by exceeding the allowable time interval for the performance of a required surveillance.

On March 17, 1995, operations personnel satisfactorily performed SP-312A, Daily Heat Power Comparison, however the thirty hour maximum interval between performances of this procedure was exceeded by 30 minutes. The cause of this event was inattention to detail by licensed personnel. Corrective actions include counselling and procedure revisions.

This report is submitted in accordance with 10CFR50.73(a)(2)(i)(B).

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TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	

TEXT (If more space is required, Use additional NRC Form 365A's (17))

EVENT DESCRIPTION:

On April 21, 1995, Florida Power Corporation's (FPC) Crystal River Unit 3 (CR-3) was in MODE ONE (POWER OPERATION), operating at 100% reactor power and generating 860 megawatts. At that time, it was determined that CR-3 had operated outside its Improved Technical Specifications (ITS) by exceeding the allowable time interval for the performance of a required surveillance.

At 0030 on March 16, 1995, CR-3 was in MODE ONE (POWER OPERATION), operating at 100% reactor power and generating 883 megawatts. Operations personnel had satisfactorily performed surveillance procedure SP-312A, Daily Heat Power Comparison. This procedure is scheduled by SP-443, Master Surveillance Plan, for performance at 0200 each day with a 25% (6 hour) window. This window provides a maximum interval of 30 hours between successive performances of this procedure.

At 0700 on March 17, 1995, operations personnel again satisfactorily performed SP-312A, however the maximum interval between performances of this procedure was exceeded by 30 minutes. At that time, operations supervisory personnel did not recognize the untimeliness of the performance of the surveillance.

On April 21, 1995, following a review of past logs it was determined that SP-312A was performed at a time other than was normally expected. Upon further review of the documentation, it was found that the performance of the procedure on March 17, 1995, was thirty minutes beyond the 25% extension of the surveillance window, as permitted by ITS Surveillance Requirement (SR) 3.0.2.

This report is submitted in accordance with 10CFR50.73(a)(2)(i)(B) for a condition prohibited by the plant's Technical Specifications.

EVENT EVALUATION

Exceeding the maximum interval for performance of this surveillance by thirty minutes did not comprise a significant safety concern. The procedure was performed satisfactorily, indicating no equipment malfunction, therefore the Reactor Protection System (RPS) would have performed as designed.

ITS SR 3.0.3 would have provided up to an additional twenty-four hours to complete the surveillance prior to declaring the applicable equipment (RPS) inoperable. The procedure was satisfactorily completed within 30 minutes following the expiration of the thirty hour maximum surveillance interval. Therefore, this event did not compromise the health and safety of the general public.

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CAUSE

The cause of this event was inattention to detail by licensed operations personnel in the failure to recognize the cumulative effect of performing SP-312A one and one-half hours earlier than nominally scheduled, by SP-443, on one shift and five hours later than nominally scheduled on the subsequent shift. The result caused the maximum allowable 30 hours interval to be exceeded by 30 minutes.

Several significant extenuating circumstances contributed to the missed surveillance. Operations personnel realized the requirement to perform the procedure within a maximum of thirty hours from its previous performance, but did not realize that it had been completed early on the previous day.

Secondly, the shift was extremely busy, due to a power reduction to 68%, necessitated by the immediate removal from service of condensate pump 1-B [SD,P](CDP-1B). The CDP-1B coupling brushes had degraded to the point requiring immediate replacement. Appropriate notifications were conducted, as required, for a greater than 15% power level change. Following replacement of the coupling brushes, CDP-1B was returned to service and power escalation was initiated.

The power escalation was halted at 90% power for the performance of quarterly portions of Performance Testing procedure, PT-325, Turbine Generator Checks, in accordance with the requirements of surveillance procedure SP-442, Special Conditions Surveillance Plan. CR-3 was not returned to full power until 0810 on March 17, 1995.

IMMEDIATE CORRECTIVE ACTION

Immediately following the determination that FPC had violated ITS Surveillance Requirement 3.3.1.2, a Problem Report was initiated.

ADDITIONAL CORRECTIVE ACTION

The Shift Supervisor, Assistant Shift Supervisor, and the Reactor Operators involved have been counseled on the importance of attention to detail.

An Operations Study Book (OSB) entry has been generated to inform all Operations personnel of the event, the importance of performing surveillances as close to the designated time frame as possible, and the need for the Shift Supervisors Log to include time of performance for SP-312A.

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ACTIONS TO PREVENT RECURRENCE

SP-312A, Daily Heat Power Comparison will be revised to include a note on the Daily Data Sheet indicating that the "daily" portion of the procedure be performed as close to 0200 as possible. If performed early or late, the Shift Supervisor should refer to the previous procedure performance to determine an allowable window.

SP-443, Master Surveillance Plan, will be reviewed to determine if any similar time-critical surveillances exist. All such procedures will have their references on the Daily Data sheets revised to clearly reflect the required performance time.

PREVIOUS SIMILAR EVENTS

There have been two previous events reported in LERs 86-012 and 87-014 involving performance of SP-312 outside the maximum surveillance window.

ATTACHMENT

Attachment 1 - Abbreviations and Acronyms

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ATTACHMENT 1

Abbreviations and Acronyms

CDP-1B	Condensate Pump 1-B
CR-3	Crystal River Unit 3
FPC	Florida Power Corporation
ITS	Improved Technical Specifications
SR	Surveillance Requirement
MODE ONE	POWER OPERATION
OSB	Operations Study Book
PT-325	Performance Testing procedure - Turbine Generator Checks
RPS	Reactor Protection System
SP-312A	Surveillance Procedure - Daily Heat Power Comparison
SP-443	Surveillance Procedure - Master Surveillance Plan