



Northeast  
Nuclear Energy

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Millstone Nuclear Power Station  
Northeast Nuclear Energy Company  
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The Northeast Utilities System  
Donald B. Miller Jr.,  
Senior Vice President - Millstone

Re: 10CFR50.73(a)(2)(i)

May 26, 1995  
ME-95-173

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Reference: Facility Operating License No. DPR-65  
Docket No. 50-336  
Licensee Event Report 95-016-00

This letter forwards Licensee Event Report 95-016-00 required to be submitted within thirty (30) days pursuant to 10CFR50.73(a)(2)(i).

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY

Donald B. Miller, Jr.  
Senior Vice President - Millstone Station

DBM/SA:dlr

Attachment: LER 95-016-00

cc: T. T. Martin, Region I Administrator  
P. D. Swetland, Senior Resident Inspector, Millstone Unit Nos. 1, 2, and 3  
G. S. Vissing, NRC Project Manager, Millstone Unit No. 2

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## LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

Millstone Nuclear Power Station Unit 2

DOCKET NUMBER (2)

05000336

PAGE (3)

1 OF 3

TITLE (4)

Fuel Handling SRO Position Assumed by Operator with Inactive SRO License

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER	
04	28	95	95	016	00	05	26	95			
THIS REPORT IS BEING SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)											
OPERATING MODE (9)		20.402(b)			20.405(c)			50.73(a)(2)(iv)			73.71(b)
POWER LEVEL (10)		20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)			73.71(c)
		20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vi)			OTHER
		20.405(a)(1)(iii)			X 50.73(a)(2)(i)			50.73(a)(2)(vii)(A)			(Specify in Abstract Below and in Text, NRC Form 366A)
		20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(vii)(B)			
		20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(iii)			

## LICENSEE CONTACT FOR THIS LER (12)

NAME

Philip J. Lutz, Nuclear Licensing

TELEPHONE NUMBER (Include Area Code)

(203) 440-2072

## COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

## SUPPLEMENTAL REPORT EXPECTED (14)

YES

(If yes, complete EXPECTED SUBMISSION DATE)

X

NO

EXPECTED  
SUBMISSION  
DATE (15)

MONTH DAY YEAR

## ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On April 28, 1995 at 1415 hours, with the plant in mode 6, the RCS temperature at 85.2°F and RCS pressure at atmospheric, an operator with an inactive SRO license assumed the position of Fuel Handling SRO for the movement of the upper guide structure into the reactor vessel. The operator had not stood the minimum number of watches in the calendar quarter to maintain his SRO license active, nor the minimum number of watches to reactivate his SRO license.

The event was caused by the operator's failure to recognize that the inactive status of his SRO license rendered him unqualified to assume the duty of Fuel Handling SRO for movement of the upper guide structure.

Corrective action consisted of:

- Preparing and conducting a shift briefing for all Operations personnel to clarify and reemphasize the license requirements of a Fuel Handling SRO.
- Revising applicable operating procedures to clearly identify the required qualifications for the Fuel Handling SRO.

This event is being reported pursuant to the requirements of 10CFR50.73(a)(2)(i), a condition prohibited by the plant's Technical Specifications.

EXPIRES: 5/31/95

**LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)  Millstone Nuclear Power Station Unit 2	DOCKET NUMBER (2)  05000336	LER NUMBER (6)			PAGE (3)
		YEAR  95	SEQUENTIAL NUMBER  — 016 —	REVISION NUMBER  00	02 OF 03

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

**I. Description of Event**

On April 28, 1995 at 1415 hours, with the plant in mode 6, the RCS temperature at 85.2°F and RCS pressure at atmospheric, an operator with an inactive SRO license assumed the position of Fuel Handling SRO. In the absence of other SRO-licensed operators, the subject SRO, acting as a work control supervisor, decided to act as Fuel Handling SRO for the movement of the upper guide structure into the reactor.

It was later determined that the subject operator did not meet the requirements of Technical Specification paragraph 6.2.2.e due to his SRO license being inactive. The operator had not maintained his SRO license active per 10CFR55.53(e) by actively standing watch as an SRO for a minimum number of shifts per calendar quarter. Also, the operator had not reactivated his SRO license per 10CFR55.53(f) by standing the minimum number of watch hours under the direction of a qualified, active-licensed SRO.

During the evolution, there was no effect on the plant and the upper guide structure was moved without incident. There were no safety systems actuated automatically or manually actuated.

**II. Cause of Event**

The event was caused by the subject SRO not being aware of the requirement for his SRO license to be active when supervising CORE ALTERATIONS. The SRO was not aware that SRO license requirements for movement of the upper guide structure are the same as for actual fuel movement.

A contributing cause was the lack of sufficient detail in operating procedures and instructions relating to SRO license requirements for performing CORE ALTERATIONS. The procedures state that an SRO must be present on the refueling floor when performing CORE ALTERATIONS but no specific requirements are spelled out for the SRO to hold an active SRO license per 10CFR55.53(e) or a reactivated SRO license or license limited to fuel handling per 10CFR55.53(f).

**III. Analysis of Event**

This event is being reported pursuant to the requirements of 10CFR50.72(a)(2)(i), a condition prohibited by the plant's Technical Specifications. Millstone Unit 2 Technical Specification 6.2.2.e states:

"ALL CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation."

The operator involved in this event possessed an SRO license but it was inactive and had not been reactivated prior to his directly supervising the movement of the upper guide structure into the reactor.

Since the operator was licensed, possessed the requisite knowledge to supervise CORE ALTERATIONS, and had no other concurrent responsibilities during the movement of the upper guide structure into the reactor, there were no safety implications resulting from this event.

**IV. Corrective Action**

Corrective action taken to prevent recurrence of the incident was to clarify and reemphasize the license requirements of the Fuel Handling SRO. This was accomplished via a shift briefing for all Operations personnel. Also, operating procedures and instructions were revised to clearly define the license requirements of SROs who directly supervise CORE ALTERATIONS.

EXPIRES: 5/31/95

**LICENSEE EVENT REPORT (LER)**  
**TEXT CONTINUATION**

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)		PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER
Millstone Nuclear Power Station Unit 2	05000336	95	- 016 -	00
				03 OF 03

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

V. Additional Information

Similar LER's: None

EIIS Code: None