

# OPERATING DATA REPORT

DOCKET NO. 50-266

DATE: 05/23/95

COMPLETED BY: M. B. Koudelka

TELEPHONE: 414 755-6480

## OPERATING STATUS

1. UNIT NAME: POINT BEACH NUCLEAR PLANT - UNIT 1
2. REPORTING PERIOD: April - 1995
3. LICENSED THERMAL POWER (MWT): 1518.5
4. NAMEPLATE RATING (GROSS MWE): 523.8
5. DESIGN ELECTRICAL RATING (NET MWE): 497.0
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 509.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 485.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:  
NA
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): NA
10. REASONS FOR RESTRICTIONS, (IF ANY):  
NA

	THIS MONTH	YEAR TO DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	719.0	2,879.0	214,607.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	341.5	2,001.0	178,840.6
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	667.3
14. HOURS GENERATOR ON LINE	329.2	1,987.9	175,700.3
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	846.9
16. GROSS THERMAL ENERGY GENERATED (MWH)	449,547	2,929,580	248,474,577
17. GROSS ELECTRICAL ENERGY GENERATED	153,290	997,210	83,935,860
18. NET ELECTRICAL ENERGY GENERATED (MWH)	144,097	951,410	80,012,061
19. UNIT SERVICE FACTOR	45.8%	69.0%	81.9%
20. UNIT AVAILABILITY FACTOR	45.8%	69.0%	82.3%
21. UNIT CAPACITY FACTOR (USING MDC NET)	41.3%	68.1%	76.5%
22. UNIT CAPACITY FACTOR (USING DER NET)	40.3%	66.5%	75.0%
23. UNIT FORCED OUTAGE RATE	0.0%	0.0%	1.5%
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): NONE			
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: NA			

DATA REPORTED AND FACTORS CALCULATED AS REQUESTED IN NRC LETTER DATED SEPTEMBER 22, 1977

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POINT BEACH NUCLEAR PLANT  
UNIT SHUTDOWNS AND POWER REDUCTIONS  
 REPORT MONTH APRIL - 1995

Docket No. 50-266  
 Unit Name Point Beach, Unit 1  
 Date May 8, 1995  
 Completed By M. B. Koudelka  
 Telephone No. 414/755-6480

No.	Date	Type <sup>1</sup>	Duration	Reason <sup>2</sup>	Method of Shutting Down	Licensee Event	System	Component	Cause and Corrective Action
2	3/11/95	S	389.8	C	1	NA	NA	NA	Scheduled refueling and maintenance (U1R22). Major work items include steam generator eddy current testing, residual heat removal rotating assembly work, channelhead blowers duct replacement, reactor vessel head shielding, fuel transfer tube shielding, G03 emergency diesel generator Phase 3B tie-in work, modify control rod drive mechanism current order timing change, degraded grid voltage relay work, steam generator blowdown heat exchanger replacement, crossover steam dump pressure switches replacement, 1A05 bus 2/3 loss of voltage logic work, component cooling small bore pipe seismic upgrades, SC-955 containment isolation valve retubing, 1RC-558 valve replacement, modify service water and component cooling supports as part of continued Bulletin 79-14 upgrades.

<sup>1</sup>F: Forced  
 S: Scheduled

<sup>2</sup>Reason:  
 A - Equipment Failure (explain)  
 B - Maintenance or Testing  
 C - Refueling  
 D - Regulatory Restriction  
 E - Operator Training & Licensing Exam  
 F - Administrative  
 G - Operational Error (explain)  
 H - Other (explain)

<sup>3</sup>Method:  
 1 - Manual  
 2 - Manual Scram  
 3 - Automatic Scram  
 4 - Continuation of Previous Shutdown  
 5 - Reduced Load  
 6 - Other (explain)

<sup>4</sup>Exhibit G - Instructions for preparation of data entry sheets LER file (NUREG-0161)

<sup>5</sup>Exhibit I - Same Source

## POINT BEACH NUCLEAR PLANT

DOCKET NO.

50-266

AVERAGE DAILY UNIT POWER LEVEL

UNIT NAME

Point Beach, Unit 1

MONTH APRIL - 1995

DATE

May 8, 1995

COMPLETED BY

M. B. Koudelka

TELEPHONE

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DAY	AVERAGE DAILY POWER LEVEL MWe NET	DAY	AVERAGE DAILY POWER LEVEL MWe NET	DAY	AVERAGE DAILY POWER LEVEL MWe NET
1	-2	11	-2	21	456
2	-2	12	-2	22	499
3	-2	13	-9	23	502
4	-2	14	-13	24	499
5	-2	15	-15	25	500
6	-2	16	-15	26	499
7	-2	17	57	27	500
8	-4	18	199	28	500
9	-8	19	371	29	496
10	-2	20	519	30	490

DOCKET NO. 50-266  
UNIT NAME Point Beach Unit 1  
DATE May 8, 1995  
COMPLETED BY M. E. Koudelka  
TELEPHONE 414/755-6480

Unit 1 operated at an average of 200 MWe net for the report period.

Licensee Event Report 50-266/95-002-00, Inadvertent Emergency Diesel Generator Start, was submitted.

Licensee Event Report 50-266/95-003-00, Inadvertent Emergency Diesel Generator Start, was submitted.

Licensee Event Report 50-266/95-004-00, Loss of Voltage Relays Not Calibrated to Technical Specification Limits, was submitted.

While performing low power physics testing, coming out of Unit 1 Refueling 22, a manual reactor trip was initiated because of dropped rods. Neither the physics procedure nor the normal operating procedure allowed a restart with the control rod configuration at that time.

Safety-related maintenance included:

1. 271/A02; 272/A01; 272/A02 Bus A01 and A02 undervoltage relay gaskets replaced.
2. 1A52-60 G01 emergency diesel generator output to 1A05 loose lug crimped.
3. 1A52-84 Bus 1A06 feed to 1X14 station service transformer overcurrent relay setpoint recalibrated.
4. 1B52-21B power to PP-13 pressurizer backup heater Group D power panel amptector retrofit installed.
5. 1C2 personnel access air lock O-rings replaced.
6. 1CV-371-O letdown line containment isolation valve diaphragm replaced.
7. 1HX-1A steam generator Furmanite removed and repair of lower south handhole.
8. HX-105A&B primary auxiliary building battery room vent cooler flushed.
9. 1MS-478 HX-1B steam generator steam header PT-478 root valve packing replaced.

10. 1MS-279 HX-18A1&2 steam generator blowdown heat exchanger outlet to T26 tank inlet throttle valve cage and gaskets replaced.
11. 1MS-2016 HX-1A steam generator steam header atmospheric steam dump control valve cage replaced.
12. 1MS-2017A HX-1B steam generator steam header non-return check valve diaphragm replaced.
13. 1MS-2054 main steam dump to condenser dump control valve trim and packing replaced.
14. 1P1B reactor coolant pump #2 seal leak corrected.
15. 1P1B-M reactor coolant pump motor return flanges installed.
16. 1RC-431C T1 pressurizer power-operated relief valve trim, packing, yoke bonnet bolts, and diaphragm replaced.
17. 1RC-570B R1 reactor vessel head vent solenoid valve control switch replaced.
18. 1RC-595 T2 pressurizer nitrogen regulator isolation valve bonnet replaced.
19. 1SC-955 reactor coolant hot leg sample isolation valve packing replaced and operator rebuilt.
20. 1SC-966B pressurizer liquid sample containment isolation valve packing replaced.
21. 1SI-839B-S Train A cold leg safety injection to test line isolation solenoid valve rebuilt.
22. 1SI-866B core deluge injection line isolation valve limit switch wiring modified.
23. 1SI-878C P15B safety injection pump R1 reactor vessel injection valve cleaned and packing adjusted.
24. 1SI-898 T13 refueling water storage tank local sample valve diaphragm replaced.
25. 1SW-726B HX-30B cavity cooling coil outlet manifold drain valve cleaned.
26. 1W1A-D1 containment accident recirculation fan accelerometers replaced.
27. Plexiglass covers were removed from 4 reactor trip breakers and anti-operation pins were installed.
28. Unit 1 containment floor liner plate inspected.

# OPERATING DATA REPORT

DOCKET NO. 50-301

DATE: 05/23/95

COMPLETED BY: M. B. Koudelka

TELEPHONE: 414 755-6480

## OPERATING STATUS

1. UNIT NAME: POINT BEACH NUCLEAR PLANT - UNIT 2 . . . . . NOTES
2. REPORTING PERIOD: April - 1995 . . . . .
3. LICENSED THERMAL POWER (MWT): 1518.5 . . . . .
4. NAMEPLATE RATING (GROSS MWE): 523.8 . . . . .
5. DESIGN ELECTRICAL RATING (NET MWE): 497.0 . . . . .
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 509.0 . . . . .
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 485.0 . . . . .
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:  
NA
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): NA
10. REASONS FOR RESTRICTIONS, (IF ANY):  
NA

	THIS MONTH	YEAR TO DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	719.0	2,879.0	199,392.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	719.0	2,733.4	174,785.0
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	233.9
14. HOURS GENERATOR ON LINE	719.0	2,652.3	172,347.0
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	302.2
16. GROSS THERMAL ENERGY GENERATED (MWH)	1,080,884	3,976,210	247,958,135
17. GROSS ELECTRICAL ENERGY GENERATED	365,940	1,342,320	84,293,430
18. NET ELECTRICAL ENERGY GENERATED (MWH)	349,498	1,280,573	80,361,088
19. UNIT SERVICE FACTOR	100.0%	92.1%	86.4%
20. UNIT AVAILABILITY FACTOR	100.0%	92.1%	86.6%
21. UNIT CAPACITY FACTOR (USING MDC NET)	100.2%	91.7%	82.5%
22. UNIT CAPACITY FACTOR (USING DER NET)	97.8%	89.5%	81.1%
23. UNIT FORCED OUTAGE RATE	0.0%	0.0%	0.9%
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): Refueling, 10/07/95, 38 days			
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: NA			

DATA REPORTED AND FACTORS CALCULATED AS REQUESTED IN NRC LETTER DATED SEPTEMBER 22, 1977

POINT BEACH NUCLEAR PLANT  
UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH APRIL - 1995

Docket No. 50-301  
Unit Name Point Beach, Unit 2  
Date May 8, 1995  
Completed By M. B. Koudelka  
Telephone No. 414/755-6480

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report No.	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause and Corrective Action To Prevent Recurrence
NA	NA	NA	NA	NA	NA	NA	NA	NA	NA

<sup>1</sup>F: Forced  
S: Scheduled

<sup>2</sup>Reason:  
A - Equipment Failure (explain)  
B - Maintenance or Testing  
C - Refueling  
D - Regulatory Restriction  
E - Operator Training &  
Licensing Exam  
F - Administrative  
G - Operational Error (explain)  
H - Other (explain)

<sup>3</sup>Method:  
1 - Manual  
2 - Manual Scram  
3 - Automatic Scram  
4 - Continuation of  
Previous Shutdown  
5 - Reduced Load  
6 - Other (explain)

<sup>4</sup>Exhibit G - Instructions  
for preparation of  
data entry sheets  
LSR file (NUREG-0161)

<sup>5</sup>Exhibit I - Same Source

## POINT BEACH NUCLEAR PLANT

AVERAGE DAILY UNIT POWER LEVELMONTH APRIL - 1995

DOCKET NO.

50-301

UNIT NAME

Point Beach, Unit 2

DATE

May 8, 1995

COMPLETED BY

M. B. Koudelka

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<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>
1	<u>486</u>	11	<u>484</u>	21	<u>486</u>
2	<u>466</u>	12	<u>488</u>	22	<u>486</u>
3	<u>487</u>	13	<u>483</u>	23	<u>488</u>
4	<u>485</u>	14	<u>486</u>	24	<u>487</u>
5	<u>486</u>	15	<u>484</u>	25	<u>486</u>
6	<u>487</u>	16	<u>486</u>	26	<u>486</u>
7	<u>486</u>	17	<u>485</u>	27	<u>487</u>
8	<u>487</u>	18	<u>486</u>	28	<u>486</u>
9	<u>486</u>	19	<u>486</u>	29	<u>486</u>
10	<u>487</u>	20	<u>486</u>	30	<u>487</u>



DOCKET NO. 50-301  
UNIT NAME Point Beach Unit 2  
DATE May 8, 1995  
COMPLETED BY M. B. Koudelka  
TELEPHONE 414/755-6480

Unit 2 operated at an average of 485 MWe net for the report period.

Licensee Event Report 50-301/95-003-00, Reactor Trip Due to Turbine Generator Electrohydraulic Control (EHC) System Oil Leak, was submitted.

Safety-related maintenance included:

1. 2B52-29B power to PP-18 pressurizer backup heater Group D power panel amptector unit installed.
2. 2MS-249 HX-1A steam generator SAX-2023 inlet sample isolation valve packing adjusted.
3. 2P2A charging pump O-rings replaced.
4. 2SI-896A&B valves cleaned of boric acid buildup.
5. Harmonic filters installed and 273/A05; 274/A05; and 275/A05 Bus A05 loss of voltage relays calibrated.

POINT BEACH NUCLEAR PLANT

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH MARCH - 1995

Docket No. 50-301  
 Unit Name Point Beach, Unit 2  
 Date May 2, 1995  
 Completed By M. B. Koudelka  
 Telephone No. 414/755-6480

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report No.	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause and Corrective Action To Prevent Recurrence
1	2/25/95	S	122.7	H	4	NA	EG	XXXXXX	The unit was maintained in hot shutdown because of seal oil inleakage into the Unit 2 generator. The generator was disassembled. A Westinghouse seal oil expert examined the seal oil package noting no problems with the component. A seal was replaced and an extra drain installed to accommodate excess oil to preclude problems with the generator.
2	3/07/95	F	14.3	A	3	301/95-003-00	HA	MECFUN	A leak in the main turbine EH control oil system caused a main steam stop valve to close. This caused an increase in reactor coolant system temperature. The unit automatically responded by reducing turbine load, followed by an overtemperature differential temperature reactor trip.

<sup>1</sup>F: Forced  
 S: Scheduled

<sup>2</sup>Reason:  
 A - Equipment Failure (explain)  
 B - Maintenance or Testing  
 C - Refueling  
 D - Regulatory Restriction  
 E - Operator Training &  
 Licensing Exam  
 F - Administrative  
 G - Operational Error (explain)  
 H - Other (explain)

<sup>3</sup>Method:  
 1 - Manual  
 2 - Manual Scram  
 3 - Automatic Scram  
 4 - Continuation of  
 Previous Shutdown  
 5 - Reduced Load  
 6 - Other (explain)

<sup>4</sup>Exhibit G - Instructions  
 for preparation of  
 data entry sheets  
 LER file (NUREG-0161)

<sup>5</sup>Exhibit I - Same Source