

LICENSEE EVENT REPORT

CONTROL BLOCK:

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME		LICENSE NUMBER										LICENSE TYPE					EVENT TYPE								
01	N	Y	N	M	P	1								4	1	1	1	1	0	3					
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28				
CATEGORY		REPORT TYPE		REPORT SOURCE		DOCKET NUMBER								EVENT DATE					REPORT DATE						
01	CON'T	-	-	L	L	0	5	0	-	0	2	2	0	1	2	1	0	7	6	0	1	0	4	7	7
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32

EVENT DESCRIPTION

02	During routine surveillance, it was discovered that 10-10-10 reactor water level																				80
03	sensors RE-18A,B, & C did not actuate within the tolerance set point band of																				80
04	<127.1 ± 2.6". A was 121.5", B was 122.0" and C was 120.0". All were in the con-																				80
05	servative direction.																				80
06	LER 76-40																				80

SYSTEM CODE		CAUSE CODE		COMPONENT CODE				PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER				VIOLATION	
07	I	B	E	I	N	S	T	R	U	N	B	O	8	0	-
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22

CAUSE DESCRIPTION

08	Apparent set point drift:																				80
09	Barton Model #288																				80
10	Serial Number 3468, 3470, and 3471																				80

FACILITY STATUS		% POWER		OTHER STATUS				METHOD OF DISCOVERY		DISCOVERY DESCRIPTION									
11	E	0	9	1					b										
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22				
FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY				LOCATION OF RELEASE											
12	Z	Z	N/A				N/A												
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22				

PERSONNEL EXPOSURES

NUMBER		TYPE		DESCRIPTION																		
13	0	0	0	Z	N/A																	
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22							

PERSONNEL INJURIES

NUMBER		DESCRIPTION																					
14	0	0	0	N/A																			
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22								

CONSEQUENCES PROBABLE

15	None																				80
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LOSS OR DAMAGE TO FACILITY

TYPE		DESCRIPTION																			
15	Z	N/A																			
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22						

PUBLICITY

17	None																				80
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8304060433 770104
PDR AD0CK 05000220
S PDR

ADDITIONAL FACTORS

18	None																				80
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NIAGARA MOHAWK POWER CORPORATION

NIAGARA MOHAWK

300 ERIE BOULEVARD, WEST
SYRACUSE N Y 13202

January 5, 1977

Mr. James P. O'Reilly
Director of Regulatory Operations
United States Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, PA. 19406

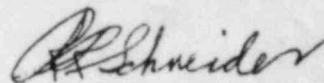
RE: Docket No. 50-220

Dear Mr. O'Reilly:

In accordance with Nine Mile Point Nuclear Station Unit #1
Technical Specification 6.9, we hereby submit Licensee Event Reports
76-40 and 76-41.

These reports were completed within the intent of the Licensee
Event Report Instruction Booklet 00E-SS-001, dated October 1974, revised
December 24, 1974. Due to the minimal significance category of these
reports, no supplemental reports are included.

Very truly yours,



R.R. Schneider
Vice President
Electric Production

TJD/mtm

Enc.

