

William E. Caldwell, Jr.
Vice President

Consolidated Edison Company of New York, Inc.
4 Irving Place, New York, N Y 10003
Telephone (212) 460-5181



May 25, 1973

Re Indian Point Unit No. 2
AEC Docket No. 50-247
Facility Operating License
DPR-26

Mr. John F. O'Leary, Director
Directorate of Licensing
U. S. Atomic Energy Commission
Washington, D. C. 20545

Dear Mr. O'Leary

5/24/73
JRH
ETW
JK
CN
JS
CWR
LB
WC

In accordance with the requirements of Technical Specification 6.6.1.B we informed you on May 16, 1973 by telegram of Abnormal Occurrence No. 3-2-4 which was identified on May 15, 1973 at 1300 Hrs. During the course of a routine plant status inspection, a leak was observed at a weld on a 3/4 inch pipe-to-socket weld branch connection between valve S-47 and line No. 355 of the Residual Heat Removal System. (The connection is located on the discharge side of the Residual Heat Removal Exchangers, upstream of the check valve 838A).

Examination of the leak indicated that it was caused by a fine crack approximately 3/4" long between the weld fillet and the half coupling socket. The location of the crack suggested that a defect internal to the weld propagated to the surface allowing leakage.

Following the discovery of the leak, the circumstances preceding the leakage were investigated. Examination of records disclosed that Vent Assembly S-47 was reworked on October 4 and 5, 1972 to replace an thinwall nipple between the socket and valve. This rework included removal of the entire vent valve assembly. A new 3/4 inch socket weld half coupling and shop fabricated assembly was welded to line 355. Both Con Edison and WEDCO Quality Control personnel inspected the welds and verified the absence of defects. The assembly was then hydrostatically tested to verify that the welds were sound.

Previous analysis as well as recent verification of such analysis indicate that stresses resulting from vibration are insufficient to cause cracking. Therefore, it is unlikely that vibration or other stresses caused the May 15, 1973 leakage. Despite that fact, the vent assembly was shortened by two inches during the recent repair. This action will reduce the possibility of vibration induced stresses in the weld.

8304060098 730525
PDR ADOCK 05000247
S PDR

50-247
inquiry

3477

John F. O'Leary

-2-

May 25, 1973

Re Indian Point Unit No. 2
AEC Docket No. 50-247
Facility Operating License
DPR-26

In addition to shortening the pipe nipple on May 16, 1973, the leaking connection was ground out, and the assembly was rewelded in accordance with the appropriate procedures. The welds were then inspected by dye penetrant and subjected to hydrostatic testing to verify their structural integrity.

The leakage resulting from this abnormal occurrence is readily detectable and is well within the makeup capabilities of the plant Chemical and Volume Control System without incurring adverse consequences to the health and safety of facility personnel or to the public. Furthermore, at the time of the occurrence the reactor was in a cold shutdown condition at atmospheric pressure and the leakage was not radioactive. For these reasons, there are no safety implications to the occurrence.

Having reviewed the circumstances of the abnormal occurrence the Con Edison Nuclear Facilities Safety Committee concurs that the occurrence does not represent a significant hazards consideration.

Very truly yours



William E. Caldwell, Jr.
Vice President

EAO/ds

Copy to James P. O'Reilly (AEC)