



Northeast  
Nuclear Energy

Rope Ferry Rd. (Route 156), Waterford, CT 06385

Millstone Nuclear Power Station  
Northeast Nuclear Energy Company  
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The Northeast Utilities System

Donald B. Miller Jr.,  
Senior Vice President - Millstone

Re: 10CFR50.73(a)(2)(i)(B)

May 22, 1995

MP-95-167

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Reference: Facility Operating License No. NPF-49  
Docket No. 50-423  
Licensee Event Report 95-008-00

This letter forwards Licensee Event Report 95-008-00 required to be submitted within thirty (30) days pursuant to 10CFR 50.73(a)(2)(i)(B).

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY

Donald B. Miller, Jr.  
Senior Vice President - Millstone Station

DBM/RLM:clc

Attachment: LER 95-008-00

cc: T. T. Martin, Region I Administrator  
P. D. Swetland, Senior Resident Inspector, Millstone Unit Nos. 1, 2, and 3  
V. L. Rooney, NRC Project Manager, Millstone Unit No. 3

9505310080 950522  
PDR ADDCK 05000423  
S PDR

## LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Millstone Nuclear Power Station Unit 3	05000423	1 OF 3

TITLE (4)
Licensed Senior Reactor Operator Supervised Refueling Operations Without Active License

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
04	23	95	95	008	00	05	22	95	FACILITY NAME	DOCKET NUMBER

OPERATING MODE (9)	6	THIS REPORT IS BEING SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)									
		20.402(b)			20.405(c)			50.73(a)(2)(iv)			73.71(b)
		20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)			73.71(c)
		20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vi)			OTHER
		20.405(a)(1)(iii)			X 50.73(a)(2)(i)			50.73(a)(2)(vii)(A)			(Specify in Abstract below and in Text, NRC Form 366A)
20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(vii)(B)					
20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)					

LICENSEE CONTACT FOR THIS LER (12)									
NAME								TELEPHONE NUMBER (Include Area Code)	
Robert L. McGuinness, Senior Engineer								(203) 447-1791 Ext. 6855	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)						EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X			NO					

## ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On April 23, 1995 at approximately 1530 hours, with the plant in Mode 6, the RCS temperature at 100 degrees Fahrenheit and pressure at 40 psia, an individual with a non-active Senior Reactor Operator (SRO) license assumed the position of refueling SRO without activating his license per the requirements of 10CFR55.53(f). This is a condition prohibited by Technical Specifications.

The event had no safety significance. The plant was in a refueling outage. The licensed SRO is highly experienced, and has maintained a license for over 10 years on the unit. Also, no problems were experienced during the event.

The corrective action at the time of the event was to inform the Shift Supervisor, who promptly dispatched an active SRO licensee to the containment to oversee the Control Rod Drive Mechanism (CRDM) unlatching operations.

Actions were taken to prevent recurrence. The Manager of Operations issued a memo to all SRO's and SS's informing them of the requirements of activating a SRO license and to have an active license prior to assuming supervision of fuel handling operations. Several SRO's were upgraded to active licensed SRO's to support fuel handling operations during the refueling outage. Also, procedures were revised to clearly define the qualification requirements for activating an SRO license for supervising refueling operations.

EXPIRES: 5/31/95

**LICENSEE EVENT REPORT (LER)**  
**TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 80.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)  Millstone Nuclear Power Station Unit 3	DOCKET NUMBER (2)  05000423	LER NUMBER (6)		PAGE (3)
		YEAR  95	SEQUENTIAL NUMBER — 008 —	REVISION NUMBER 00

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

I. Description of Event

On April 23, 1995 at approximately 1530 hours, with the plant in Mode 6, the RCS temperature at 100 degrees Fahrenheit and pressure at 40 psia, a licensed Senior Reactor Operator (SRO) had assumed the position of refueling SRO without having his license activated per the requirements of 10CFR55.53(f). The plant was in the process of shutting down for a refueling outage.

Earlier, at approximately 1500 hours the SS requested that the subject SRO provide fuel handling SRO coverage to supervise the Control Rod Drive Mechanism (CRDM) unlatching operations.

Prior to assuming his responsibilities, the subject SRO discussed his qualifications with three Operations SS's. The subject SRO is the Supervisor of Reactor Engineering, and has maintained an SRO license on the Unit since 1985, a year before the plant received the Operating License. He was informed that the Supervisor of Operator Training (an SRO) was also contacted, and they had determined that he was qualified as a SRO for fuel handling. The subject SRO concluded that he had adequate training, authority, and knowledge to work as the fuel handling SRO. Accordingly, he assumed the fuel handling SRO duties at approximately 1530 hours. At 1702 hours, the SRO was requested to stop the CRDM unlatching operations due to a concern over his qualifications. The Shift Supervisor promptly dispatched an active SRO licensee to the containment to oversee the remainder of the CRDM unlatching operations.

II. Cause of Event

The cause of the event was a programmatic weakness, in that a number of Operations SS's, the Supervisor of Operator Training (SRO), and the Supervisor of Reactor Engineering (SRO) were not adequately aware that the requirements in 10CFR55.53(f)(1) and (2) for reactivating a SRO license, are also applicable prior to SRO supervision of refueling operations.

III. Analysis of Event

This event is reported in accordance with the requirements of 10CFR50.73(a)(2)(i), as a condition prohibited by Technical Specifications. Technical Specification 6.2.2.e states that "ALL CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Reactor Operator or licensed Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation." CORE ALTERATIONS include CRDM unlatching operations. The individual is a licensed SRO who is highly capable. However, his SRO license was not activated in accordance with the requirements of 10CFR55.53(f), prior to directly supervising the CRDM unlatching operations. The event had no safety significance. The plant was in a refueling outage. No CRDM latching problems were experienced during this time.

IV. Corrective Action

The immediate action taken was to inform the Shift Supervisor, who promptly dispatched an active SRO licensee to the containment to oversee the remainder of the CRDM unlatching operations.

Several corrective actions were taken to prevent recurrence. The Manager of Operations issued a memo to all SRO's and SS's informing them of the requirements of activating a SRO license and to have an active license prior to assuming supervision of fuel handling operations. Several additional individuals were upgraded to active licensed SRO's to support fuel handling operations during the refueling outage. Also, procedures were revised to clearly define the qualification requirements for activating an SRO license for supervising refueling.

EXPIRES: 5/31/95

# LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION  
COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING  
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BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION,  
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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
Millstone Nuclear Power Station Unit 3	05000423	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	03 OF 03
		95	— 008 —	00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

## V. Additional Information

There have been no similar Licensee Event Reports involving Licensed Reactor Operator responsibilities.

## EIIS Codes

EIIS codes are not applicable to this event.