

LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

/0/1/ /V/A/N/A/S/1/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

/0/1/ REPORT /L/ (6) /0/5/0/0/0/3/3/8/ (7) /0/8/0/8/8/1/ (8) /0/4/1/8/8/4/ (9)
SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On August 8, 1981, with Unit 1 at 100 percent power, the "B" Reactor Trip Breaker/
/0/3/ / failed to open when a high pressurizer pressure signal was simulated. The "A" /
/0/4/ / reactor trip breaker was available in the event of an automatic reactor trip; /
/0/5/ / therefore the health and safety of the general public were not affected. This /
/0/6/ / is reportable pursuant to T.S. 3.3.1.1 and 6.9.1.9.b. /
/0/7/ / /
/0/8/ / /

SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
/0/9/ /1/A/ (11)	/E/ (12)	/B/ (13)	/C/K/T/B/R/K/ (14)	/A/ (15)	/Z/ (16)
LER/RO	EVENT YEAR	SEQUENTIAL REPORT NO.	OCCURRENCE CODE	REPORT TYPE	REVISION NO.

(17) REPORT NUMBER /8/1/ /-/ /0/6/3/ /-/ /0/3/ /X/ /-/ /2/

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
/A/ (18)	/Z/ (19)	/B/ (20)	/Z/ (21)	/0/0/0/0/ (22)	/Y/ (23)	/N/ (24)	/N/ (25)	/W/1/2/0/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / A metal latch in the undervoltage trip attachment failed which prevented the /
/1/1/ / trip signal from opening the "B" Reactor Trip Breaker. The manual reactor trip /
/1/2/ / switches in the Control Room and locally were not affected. The undervoltage /
/1/3/ / trip attachment was replaced and the reactor trip breaker retested. /
/1/4/ / /

FACILITY STATUS	%POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
/1/5/ /E/ (28)	/1/0/0/ (29)	/ NA / (30)	/D/ (31)	/ Operator Observation /

ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
/1/6/ /Z/ (33)	/Z/ (34)	/ NA /	/ NA /

PERSONNEL EXPOSURES NUMBER	TYPE	DESCRIPTION (39)
/1/7/ /0/0/0/ (37)	/Z/ (38)	/ NA /

PERSONNEL INJURIES NUMBER	DESCRIPTION (41)
/1/8/ /0/0/0/ (40)	/ NA /

LOSS OF OR DAMAGE TO FACILITY TYPE	DESCRIPTION (43)
/1/9/ /Z/ (42)	/ NA /

PUBLICITY ISSUED	DESCRIPTION (45)	NRC USE ONLY
/2/0/ /N/ (44)	/ NA /	/ / / / / / / / / / / / /

NAME OF PREPARER E. Wayne Harrell PHONE (703) 894-5151

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PDR ADOCK 05000338
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Updated Report - Previous Report Date 09-16-81

Virginia Electric and Power Company
North Anna Power Station, Unit No. 1
Docket No. 50-338
Report No. LER 81-063/03X-2

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Description of Event

On August 8, 1981, with Unit 1 at 100% power, the "B" Main Reactor Trip Breaker failed to open when a high pressurizer pressure signal was simulated.

Probable Consequences of Occurrence

The failure of the undervoltage trip attachment would prevent the "B" Main Reactor Trip Breaker from opening for any automatic reactor trip signal. The manual reactor trip switches were not affected by the failure of the undervoltage trip device. The "A" Reactor Trip Breaker was available in the event of any automatic reactor trip; therefore, the health and safety of the general public were not affected.

Cause of Event

A metal latch in the undervoltage trip attachment failed. This device is a solenoid that is de-energized by an automatic reactor trip signal. The solenoid plunger normally impacts a trip bar inside the reactor trip breaker switchgear. The failed latch prevented the plunger linkage from contacting the trip bar in the "B" Main Reactor Trip Breaker.

Immediate Corrective Action

The unit began a rampdown in order to be in hot standby within the 6 hour Action Statement of T.S. 3.3.1.1. The undervoltage trip attachment was replaced and the trip breaker was retested satisfactorily. The rampdown was stopped at approximately 92% power and the Action Statement was cleared within two hours of the failure discovery.

Scheduled Corrective Action

No further corrective action is required.

Action Taken To Prevent Recurrence

No further action is required.

Generic Implications

This is the only failure of this type that has been reported to Westinghouse. This is an isolated event and has no generic implications.



VIRGINIA ELECTRIC AND POWER COMPANY

NORTH ANNA POWER STATION

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MINERAL, VIRGINIA 23117

April 18, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
016 Phillips Building
Washington, D.C. 20555

Serial No. N-81-150A
NO/DAH: 11
Docket No. 50-338
License No. NPF-4

Dear Sirs:

Pursuant to North Anna Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following Update License Event Report applicable to North Anna Unit No. 1.

Report No.

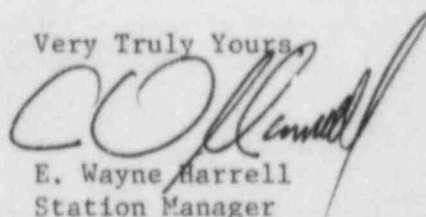
Applicable Technical Specifications

LER 81-063/03X-2

T.S. 6.9.1.9.b

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very Truly Yours,



E. Wayne Harrell
Station Manager

Enclosures (3 copies)

cc: Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

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