LICENSEE EVENT REPORT

CONTROL BLOCK: [ ] (1–6)

(PLEASE PRINT ALL REQUIRED INFORMATION)

	LICENSEE NAME	LICENSE NUMBER	LICENSE TYPE	EVENT TYPE
01	NYREG 1 (7–14)	00-00000-00 (15–25)	4 1 1 1 1 (26–30)	0 3 (31–32)

	CATEGORY	REPORT TYPE	REPORT SOURCE	DOCKET NUMBER	EVENT DATE	REPORT DATE
01 CON'T	M I (57–58)	L (59)	L (60)	0 5 0 – 0 2 4 4 (61–68)	0 9 0 2 7 5 (69–74)	0 9 2 9 7 5 (75–80)

EVENT DESCRIPTION

02 During performance of measures to eliminate erratic steam generator (SG) level indica-
03 tion and control signal to feedwater control, level signal input was transferred from
04 LT-461 to LT-462. Further investigation indicated this constituted non-compliance
05 with IEEE Standard 279-1971. (Unusual Event 50-244/75-06)
06

	SYSTEM CODE	CAUSE CODE	COMPONENT CODE	PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER	VIOLATION
07	I A (10)	F (11)	I N S T R U (12–17)	N (43)	F 1 8 0 (44–47)	N (48)

CAUSE DESCRIPTION

08 This standard was not properly considered during PORC review of EM-142 Level Trans-
09 mitter LT-462 Installed In Place of LT-461 to Control LM-463F and LC-480. The IEEE
10 protection criteria stipulates that when an equipment failure (cont'd. on attached sheet)

	FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
11	E (9)	1 0 0 (10–12)	NA (13–44)	d (45)	Receipt of letter from N (46–80)

	FORM OF ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
12	Z (9)	Z (10)	NA (11–44)	NA (45–80)

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION
13	0 0 0 (9–11)	Z (12)	NA (13–80)

PERSONNEL INJURIES

	NUMBER	DESCRIPTION
14	0 0 0 (9–11)	NA (12–80)

OFFSITE CONSEQUENCES

15 NA

LOSS OR DAMAGE TO FACILITY

	TYPE	DESCRIPTION
16	Z (9)	NA (10–80)

PUBLICITY

17 NA

ADDITIONAL FACTORS

18 NA

19

NAME: Bruce A. Snow PHONE: 716/546-2700, ext. 291-214

Cause Description (cont'd.)

in a control system can cause a condition requiring protective action and can also prevent the proper operation of a portion of that condition's protection system, then the remaining operational channels of the protection system must be capable of providing adequate protection in the event of a second random failure. LT-462, as a result of the circuitry change, was made part of the level control system, the Lo-Lo water level protection system, and the Lo S/G feedwater reactor trip logic. The investigation showed that the protection system was not able to provide adequate reactor trip protection in the event of a failure in LT-462 and with a second random failure, as required by the IEEE criteria.

The Committee discussed the results of the investigation and reviewed the status of steam generator level instruments with respect to the Technical Specification, Table 3.5-1. It was determined that loss of one level channel did not establish conditions less limiting than the limiting conditions required; however the PORC recommended to return level control to original status to restore conformance to the IEEE standard. This was accomplished on 9/2/75 using procedure EM-148, "Level Transmitter LT-461 to Control LT-463F and LC-480 in Place of Level Transmitter LT-462."

ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649

LEON D. WHITE, JR.
VICE PRESIDENT

TELEPHONE
AREA CODE 716 546-2700

September 29, 1975

Mr. James P. O'Reilly, Director
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region 1
631 Park Avenue
King of Prussia, Pennsylvania 19406

Subject: Unusual Event 75-06, Transfer of "A" Steam Generator level control input signal from level transmitter LT-461 to level transmitter LT-462
R. E. Ginna Nuclear Power Plant, Unit No. 1
Docket No. 50-244

Dear Mr. O'Reilly:

In accordance with Technical Specifications, Article 6.6.2b, the attached report of Unusual Event 75-06 is hereby submitted. Two additional copies of this letter and the attachment are enclosed.

Very truly yours,

L. D. White, Jr.

Attachment

cc: Mr. Donald F. Knuth (30)
Mr. William G. McDonald (3)

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