NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
JAMES A. FITZPATRICK NUCLEAR POWER PLANT	0 5 0 0 0 3 3 3	1 OF 0 2

TITLE (4)
REACTOR TRIP DUE TO LOW WATER LEVEL

EVENT DATE (5)			LER NUMBER (6)					REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 3	2 5	8 4	8 4	—	0 1 0	—	0 0	0 4	1 9	8 4		0 5 0 0 0
												0 5 0 0 0

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 2 5	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	X 50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
ROBERT T. LISENO, MAINTENANCE SUPERINTENDENT	AREA CODE 3 1 5 — 3 4 2 - 3 8 4 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
A	B J	C O N D	G 2 1 0	Y							
A	S J	P	W 3 1 8	N							

SUPPLEMENTAL REPORT EXPECTED (14)

		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While operating at 25% power during a reactor startup a scram occurred caused by low reactor vessel level. The low level was caused by the loss of the single operating reactor feed pump due to a control oil leak. HPCI and RCIC were manually initiated and used to maintain vessel level. After the need for HPCI had passed it was noted that a gasket on the gland seal condenser had developed a leak. To isolate the leak, HPCI was made inoperative. The transient proceeded normally and all systems performed as designed. No significant hazard to the public health and safety existed.

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

While operating at 25% power during a reactor startup a scram occurred caused by low reactor vessel level. The low level was caused by the loss of the single operating feed pump. Vessel level decreased by 1235 inches above top of active fuel. HPCI and RCIC were manually initiated to control vessel level. The primary containment isolation system functioned as designed.

Subsequent inspection of the A reactor feed pump revealed the high pressure line supplying control oil to the two turbine stop valves disconnected in the oil sump. This connection is referenced as an assembly of parts 36, 28, 55 and 56 on General Electric Drawing 509E155FY in GEK-31991. It was determined that this fitting was incorrectly assembled during the 1983 refueling and over a period of time vibrated loose. With this line disconnected the hydraulic oil in the HP and LP steam valve actuators was dumped to the oil sump resulting in a loss of control oil pressure. The loss of oil pressure resulted in a loss of control oil pressure. The loss of oil pressure resulted in closure of the stop valves. This connection was reassembled with the proper crimp on the internal ferrule and put back into operation. The reactor feed pumps will be considered for inclusion in an expanded QA program for balance of plant equipment.

During the transient a gasketed flange on the HPCI gland seal condenser developed a large leak (refer to LER 84-009-00 for a similar occurrence). The leak did not affect the operability of the HPCI system. When the need for HPCI had passed, HPCI was removed from service and the gasket was replaced on the gland seal condenser. The torque on flange bolts was increased in accordance with manufacturers instructions over that previously used. The metal band suggested by SIL-129 could not be installed since the new gasket protruded past the edge of the flange and could not be trimmed.

Since plant systems operated as designed, this transient did not represent a significant hazard to the public's health and safety.

James A. FitzPatrick
Nuclear Power Plant
P.O. Box 41
Lycoming, New York 13093
315 342.3840

Corbin A. McNeill, Jr.
Resident Manager

New York Power Authority

April 19, 1984
JAFP 84-0430

Document Control Desk
United States Nuclear Regulatory Commission
Washington, DC 20555

REFERENCE: DOCKET NO. 50-333
LICENSEE EVENT REPORT: 84-010-00

Dear Sir:

We have enclosed the referenced Licensee Event Report in accordance with 10CFR50.73.

If there are any questions concerning this report, please contact Mr. Robert Liseno at 315-342-3840, extension 220.

Very truly yours,

by dir. R Cameron

CORBIN A. McNEILL, JR.
RESIDENT MANAGER

CAM:RTL:nan
Enclosure

CC: Regional Administrator (1)
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NRC Resident Inspector
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LER/OR File

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