

LICENSEE EVENT REPORT

CONTROL BLOCK:

1					6

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME					LICENSE NUMBER					LICENSE TYPE					EVENT TYPE								
01	I	L	D	R	S	2	0	0	-	0	0	0	0	-	0	0	4	1	1	1	1	0	3
7	8	9				14	15								25	26						31	32

CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER					EVENT DATE					REPORT DATE										
01	CONT		L	L	0	5	0	-	0	2	3	7	0	4	1	3	7	6	0	5	1	3	7	6
7	8	57	58	59	60	61						68	69						74	75				80

[illegible]

012	DURING FUEL LOADING ON UNIT-2, ALL PERSONNEL WERE NOT EVACUATED FROM	7	8	9	80
003	THE 613' LEVEL (WHICH IS WITHIN LINE-OF-SIGHT OF THE REACTOR VESSEL GRID) FOR	7	8	9	80
004	CONTROL ROD EXERCISING. FAILURE TO VERIFY PERSONNEL EVACUATION DURING CONTROL ROD	7	8	9	80
005	MOVEMENTS REPRESENTED A VIOLATION OF DRESDEN PROCEDURE DFP 800-1 SECTION D.4	7	8	9	80
006	A SIMILAR EVENT OCCURRED ON 4-21-75 (REPORT NO. 50-237/1975-27). FOLLOWING	7	8	9	80
	SYSTEM CAUSE	PRIME COMPONENT	COMPONENT	(SEE ATTACHED SHEET)	80

SYSTEM CODE 07 CAUSE CODE ZZ COMPONENT CODE CONROD PRIME COMPONENT SUPPLIER W COMPONENT MANUFACTURER G080 VIOLATION Y (SEE ATTACHED SHEET)

CAUSE DESCRIPTION

08	THE EVENT WAS CAUSED BY A VIOLATION OF EXISTING STATION PROCEDURES. IT	80
09	HAS BEEN ESTABLISHED THAT OPERATIONS INFORMED THE 613 ELEVATION OF THE UPCOMING	80
10	CONTROL ROD EXERCISE; HOWEVER, PERSONNEL NOT DIRECTLY INVOLVED IN FUEL HANDLING	80
FACILITY		METHOD OF
		(SEE ATTACHED SHEET)

FACILITY STATUS: 11 H % POWER: 000 OTHER STATUS: NA METHOD OF DISCOVERY: A DISCOVERY DESCRIPTION: NA (SEE ATTACHED SHEET) 80

FORM OF ACTIVITY RELEASED 2 CONTENT OF RELEASE 2 AMOUNT OF ACTIVITY NA LOCATION OF RELEASE NA

PERSONNEL EXPOSURES

NUMBER				TYPE	DESCRIPTION
13	0	0	0	Z	NA

PERSONNEL INJURIES

NUMBER				DESCRIPTION	
1	4	0	0	0	NA

OFFSITE CONSEQUENCES

15	NA
7 8 9	

LOSS OR DAMAGE TO FACILITY

TYPE			DESCRIPTION
16	Z		NA

PUBLICITY

17	NA
7 8 9	8304050151 310015

ADDITIONAL FACTORS

18	NA	S	PDR
7 8 9			

19

7 8 9

NAME: R.W. COEN PHONE: EXT. 443

EVENT DESCRIPTION (continued)

this event, the 613 reactor building elevation was evacuated and the doors were locked prior to any control rod movements. (50-237/1976-25)

CAUSE DESCRIPTION (continued)

were not instructed to evacuate the area. Action will be taken to reemphasize to responsible personnel the importance of adhering to procedures, not only insofar as the procedures apply to one's work group, but as they may apply to any person in the work area.



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

Louham

BBS LIR #380-76

May 13, 1976

Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137



Enclosed please find Reportable Occurrence number 50-237/1976-25.
This report is being submitted to your office in accordance with the
Dresden Nuclear Power Station Technical Specifications, Section 6.6.B.

B. B. Stephenson
B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:smp

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

5495

COPY SENT REGION *III*

[illegible]

CHARGE-OUT SHEET

[illegible]