

LICENSEE EVENT REPORT

CONTROL BLOCK 1 6

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME		LICENSE NUMBER										LICENSE TYPE					EVENT TYPE							
01	N	Y	R	E	G	I	0	0	-	0	0	0	0	-	0	0	4	1	1	1	1	0	3	
7	8	9				14	15									25	26					30	31	32

CATEGORY		REPORT TYPE		R. PORT SOURCE		DOCKET NUMBER										EVENT DATE					REPORT DATE					
01	CON'T			L	L	0	5	0	-	0	2	4	4	0	1	1	5	7	6	0	2	0	6	7	6	
7	8		57	58	59	60	61						68	69							74	75				80

EVENT DESCRIPTION

02 During routine 105 psig hydro test of residual heat removal (RHR) suction piping from CV
 03 sump B to reactor coolant drain tank pumps, 2.85 gal./hr. was measured at "A" pump
 04 suction flange. Flange bolts were tightened and leakage was reduced to zero.
 05 (Reportable Occurrence 76-03, 30-day)
 06

SYSTEM CODE		CAUSE CODE		COMPONENT CODE					PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER					VIOLATION	
07	W	G	A	P	I	P	E	X	X	N	Z	9	9	9	Y		
7	8	9	10	11	12					17	43	44		47	48		

CAUSE DESCRIPTION

08 Maintenance personnel had tightened flange bolts after maintenance for proper perform-
 09 lance at normal operating pressure, but had failed to do so sufficiently to withstand test
 10 pressure. The PORC recommended that maintenance (cont'd. under Additional Factors)

FACILITY STATUS		% POWER		OTHER STATUS					METHOD OF DISCOVERY		DISCOVERY DESCRIPTION				
11	E	0	5	0	NA	NA	b	NA							
7	8	9	10	11	12	13	44	45	46				80		

FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY					LOCATION OF RELEASE				
12	Z	Z	NA	NA	NA	NA	NA	NA					
7	8	9	10	11			44	45				80	

PERSONNEL EXPOSURES

NUMBER		TYPE		DESCRIPTION				
13	0	0	0	Z	NA			
7	8	9	11	12	13			

PERSONNEL INJURIES

NUMBER		DESCRIPTION				
14	0	0	0	NA		
7	8	9	11	12		

OFFSITE CONSEQUENCES

15	NA
7	8

LOSS OR DAMAGE TO FACILITY

TYPE		DESCRIPTION				
16	Z	NA				
7	8					

PUBLICITY

17	NA
7	8

8304050146 760206
 PDR ADOCK 05000244
 S PDR

ADDITIONAL FACTORS

18 (cont'd. from Cause Description) procedure used for inspection of pump would include
 19 verification that flanges were properly tightened.

NAME: Charles V. Hartlieb

PHONE: 716/546-2700, ext. 291-216



ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649

LEON D. WHITE, JR.
VICE PRESIDENT

TELEPHONE
AREA CODE 716 541-2700

February 6, 1976



Mr. James P. O'Reilly, Director
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

Subject: Reportable Occurrence 76-03 (30-day report) Residual Heat
Removal System low pressure piping excessive leakage.
R. E. Ginna Nuclear Power Plant, Unit No. 1
Docket No. 50-244

Dear Mr. O'Reilly:

In accordance with Technical Specifications, Article 6.9.2b, the attached report of Reportable Occurrence 76-03, 30-day, is hereby submitted. Two additional copies of this letter and the attachment are enclosed.

Very truly yours,

L. D. White, Jr.
L. D. White, Jr.

Attachment

cc: Mr. John G. Davis (30)
Mr. William G. McDonald (3)

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