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December 13, 1976

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U.S. NUCLEAR REGULATORY COMMISSION
Mail Section

Director of Nuclear Reactor Regulation
Attn: Mr. Dennis L. Ziemann, Chief
Operating Reactors - Branch 2
Division of Operating Reactors
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Dresden Station Unit 2
High Pressure Coolant Injection (HPCI)
System Flow Testing
NRC Docket No. 50-237

RO?

Reference (a): Dresden Station Reportable
Occurrence No. 50-237/76-66.

Dear Mr. Ziemann:

Reference (a) reported the failure of the high pressure coolant injection (HPCI) system injection valve 2-2301-8. In order to maintain the HPCI system operability, this valve has been placed in the open position. Although this line-up maintains the injection capability of the HPCI system, it makes more difficult the complete surveillance flow testing.

Section 4.5.C.1 requires flow testing at reactor vessel pressure between 1150 and 150 psig. Although no interval is specified for this flow testing in the Technical Specifications, this flow testing has been conducted routinely on a quarterly basis. The flow test conducted during this quarter included the usual run at 900 psig vessel pressure but the 1150 psig test was deferred. This test was not conducted at this time in order to avoid the possibility of injecting the HPCI into the vessel. The HPCI water supply is from the storage tank and is cold, oxygenated water. Although of reactor quality in terms of its chemistry, it is not desirable to inject this water into the vessel under operating conditions considering feedwater thermal cycles, accelerated corrosion due to oxygenated water, and the potential for a flux spike.

Conducting the flow test at 900 psig provides adequate assurance of the HPCI operability when the measured parameters of pump flow, pump pressure, and turbine speed are compared to previous test results. This shows that the pump is operating on its established performance curve.

Past operating history reveals no significant deterioration of the pump and turbine performance since 1970, certainly no deterioration

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is to be expected in the interim period until full pressure testing is resumed.

Repairs of the 2-2301-8 motor operated valve is to be completed at the next outage of approximately three days duration which will occur in any case in late April at a hydraulic snubber inspection. This means that possibly one additional abbreviated flow test will be conducted prior to complete the restoration of the system on our present quarterly flow testing schedule.

One (1) signed original and 39 copies are provided for your use.

Very truly yours,

G. A. Abrell
Nuclear Licensing Administrator
Boiling Water Reactors