

January 8, 1974

Re: Indian Point Unit No. 2
Facility Operating License
DPR-26
A.O. 4-2-1

Mr. James P. O'Reilly, Director
Regulatory Operations, Region I
U. S. Atomic Energy Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Dear Mr. O'Reilly,

The following report is provided pursuant to the requirements of Section 6.12.2(a) of the Technical Specifications to Facility Operating License No. DPR-26.

An analysis of the results of monthly periodic surveillance test PT-M11 (Steam Line Pressure Analog Channel Functional Test) indicated that the setting of one of the four steam line pressure bistables associated with high steam flow protection was below that specified in Table 3-1 of the Facility Technical Specification. Item 5 in this table specifies the setting limit to be equal to or greater than 600 psig. The setting found for this particular bistable was approximately 568 psig. The other three bistables were found to be set correctly.

At the time this test was conducted the reactor was in the cold shutdown condition for maintenance work associated with repairs to the feedwater piping for No. 22 steam generator. The bistable found out of calibration has been reset correctly and the channel associated with this device retested satisfactorily.

As stated in our letters to Mr. O'Leary, Director, Directorate of Licensing, of December 28, 1973 and January 4, 1974, our Engineering Department is conducting an investigation into the cause of the trouble with this and other set points.

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Mr. James P. O'Reilly, Director

Mr. Bert Davis of your office was notified of this occurrence by Mr. John Makepeace by telephone on January 4, 1974.

Very truly yours,

Warren R. Cobean, Jr.

Warren R. Cobean, Jr., Manager
Nuclear Power Generation

cc: Mr. John P. O'Leary