

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 2 3 4 5 6

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME										LICENSE NUMBER										LICENSE TYPE										EVENT TYPE									
01 ILDRS2										00-000000-00										41111										01									
7 8 9 14										15 25										26 30										31 32									

CATEGORY										REPORT TYPE										REPORT SOURCE										DOCKET NUMBER										EVENT DATE										REPORT DATE									
01 CONT										T										L										050-0237										052576										060476									
7 8 57 58										59 60										61 68										69 74										75 80																			

EVENT DESCRIPTION

02 AT 0251 HOURS ON 5-25-76, DURING AUTOMATIC BLOWDOWN SURVEILLANCE TESTING, THE 203-3A																																																																																80
03 RELIEF VALVE REMAINED OPEN, CAUSING A CONTINUOUS BLOWDOWN CONDITION. BY 0300 HOURS, TORUS																																																																																80
04 LEVEL HAD RISEN TO MINUS 1.2 INCHES, A TECH SPEC VIOLATION, WHEN IT BECAME EVIDENT																																																																																80
05 THAT THE VALVE COULD NOT BE CLOSED (AT 0303 HOURS), A MANUAL SCRAM OF THE UNIT WAS																																																																																80
06 INITIATED. AT 0310 HOURS, FULL-FLOW TORUS COOLING WAS INITIATED. MAXIMUM TRANSIENT																																																																																80
(SEE ATTACHED SHEET)																																																																																80

SYSTEM CODE										CAUSE CODE										COMPONENT CODE										PRIME COMPONENT SUPPLIER										COMPONENT MANUFACTURER										VIOLATION									
07 SF										E										VALVOP										N										T020										Y									
7 8 9 10										11										12 17										43										44 47										48									

CAUSE DESCRIPTION

08 THE FAILURE WAS CAUSED BY EXCESSIVE LEAKAGE ON THE PILOT STAGE OF THE VALVE AS DETER-																																																																																80
09 MINED BY THE VENDOR'S TEST PROCEDURE 8.1. TO CORRECT THIS DEFICIENCY, BOTH THE																																																																																80
10 PILOT STAGE AND SECONDARY STAGE OF THE VALVE WERE REPLACED. THIS ACTION RETURNED THE																																																																																80
(SEE ATTACHED SHEET)																																																																																80

FACILITY STATUS										% POWER										OTHER STATUS										METHOD OF DISCOVERY										DISCOVERY DESCRIPTION									
11 C										015										NA										B										NA									
7 8 9										10 12 13										44 45										46 80																			

FORM OF ACTIVITY RELEASED										CONTENT OF RELEASE										AMOUNT OF ACTIVITY										LOCATION OF RELEASE									
12 Z										Z										NA										NA									
7 8 9										10 11										44 45										80									

PERSONNEL EXPOSURES

NUMBER										TYPE										DESCRIPTION									
13 000										Z										NA									
7 8 9 11										12 13										80									

PERSONNEL INJURIES

NUMBER										DESCRIPTION									
14 000										NA									
7 8 9 11										12 80									

OFFSITE CONSEQUENCES

15 NA																																																																																80
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LOSS OR DAMAGE TO FACILITY

TYPE										DESCRIPTION									
16 Z										NA									
7 8 9 10										80									

PUBLICITY

17 NA																																																																																80
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ADDITIONAL FACTORS

18 NA																																																																																80
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19																																																																																80
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NAME: ROBERT W. COEN

PHONE: EXT. 443

EVENT DESCRIPTION (continued)

conditions with respect to the torus were as follows: at 0340 hours, torus water temperature was 98 degrees F; at 0430 hours, torus level was plus 2.25 inches. During the event, peak drywell pressure was 0.32 psi; peak torus pressure was 0.42 psi. Following the unit shutdown, a primary containment entry was made to visually inspect the torus and to troubleshoot valve 203-3A. No anomalies were noted during either the internal or the external torus inspection. By 0638 hours, torus level was again within Tech Spec limits. This is the first failure of a Target Rock safety/relief valve at Dresden. (50-237/1976-34)

CAUSE DESCRIPTION (continued)

valve leakage to within the vendor's specifications. To further ensure valve operability, the solenoid valve was disassembled, inspected, and replaced following the discovery of foreign matter in the solenoid valve body. After repairs had been completed, the unit was returned to operation on May 26, and the valve was successfully tested at both 150 and 950 psi.

To prevent recurrence, the pilot stage of these valves will be leak-tested during the annual refueling outages. Valve 2-203-3A is a 6-x10- inch combination safety/relief valve, model 7367, manufactured by Target Rock Corp.



Commonwealth Edison

Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

BBS Ltr. #439-76

June 7, 1976



Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Enclosed please find Reportable Occurrence number 50-237/1976-34.
This report is being submitted to your office in accordance with the
Dresden Nuclear Power Station Technical Specifications, Section 6.6.B.

B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:smp

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

5943

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