NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
LaSalle County Station Unit 1	0 5 0 0 0 3 7 3	1 OF 0 3

TITLE (4)
RWCU Differential Flow Isolation Calibration

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 3	2 0	8 4	8 4	—	0 1 8	— 0 0	0 4	1 8	8 4	LaSalle County 2	0 5 0 0 0 3 7 4
											0 5 0 0 0

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 2 0	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	X 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
M. Cray, extension 279	AREA CODE 8 1 5 3 5 7 - 6 7 6 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
D	C E	Z 9 9 9	Z 9 9 9	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While investigating a differential flow indication problem it was discovered that incorrect calibration data had been used to calibrate the Reactor Water Cleanup System (CE) inlet flow transmitter. The cause was due to conflicting instrument data sheets. The RWCU System in Unit 1 was isolated per the Technical Specifications. Unit 2 was in Cold Shutdown. The instrument calibration procedures were revised and the instruments were recalibrated to the correct values. The conflicting data sheets will be revised.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

I. EVENT DESCRIPTION

On March 20, 1984, while investigating the cause of Reactor Water Cleanup System (RWCU, CE) differential flow reading downscale (Work Request L34607), it was discovered that the wrong data had been used to set the span of the Reactor Water Cleanup System Inlet Flow Transmitter 1E31-N503. This resulted in an indicated differential flow less than actual differential flow and a RWCU System isolation (JM) setpoints were higher than the LCO setpoint of 87.5 G.P.M. This condition has existed in both units since their respective licenses were issued. The Unit 1 RWCU System was isolated per Action 22 of Technical Specification Table 3.3.2-1. Unit 2 was in Cold Shutdown.

II. CAUSE

The cause of the wrong data being used was that when the calibration procedure LIS-RT-01 was written, the data used was from Sargent & Lundy Data Sheet FT25 for flow transmitters 1E31-N503 (Unit 1) and 2E31-N503 (Unit 2), which states that 0-94 inches water column equals 0-400 G.P.M. Sargent & Lundy Data Sheet FE56 (for the flow element which feeds the flow transmitters) also was supplied, but not referenced in the procedure preparation, which states that 120 inches water column equals 500 G.P.M. When calculated, using this information, 400 G.P.M. would then equate to 76.8 inches water column.

III. PROBABLE CONSEQUENCES OF THE OCCURRENCE

The isolation would have occurred at a value slightly higher than Technical Specification required values. Other RWCU leak detection methods for isolation were available at the time.

IV. CORRECTIVE ACTIONS

Investigation into the flow difference resulted in the discovery of the data sheet conflict. Additional checking discovered, per print N73927 of Specification J-2961, that Unit 1 and Unit 2 Flow Elements (G33-N504) had different values for span. LIS-RT-101 and LIS-RT-201 were revised to reflect the proper spans. The Station Nuclear Engineering Department (SNED) was contacted to determine which data sheet was correct. It was determined that the numbers on Drawing J-2961-N73927 were correct. Sargent & Lundy data sheets are being corrected to reflect this determination. (Action Item Record 01-84-67060).

The instruments in question were recalibrated on March 21, 1984 for Units 1 and 2.

V. PREVIOUS OCCURRENCES

None.

VI. NAME AND TELEPHONE NUMBER OF PREPARER

Mike Cray (815)357-6761, extension 276.

Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

April 18, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Reportable Occurrence Report #84-019-00, Docket #050-373 is being submitted to your office in accordance with 10 CFR 50.73.

G. J. Diederich
Superintendent
LaSalle County Station

GJD/MLD/kg

Enclosure

xc: NRC, Regional Director
INPO-Records Center
File/NRC

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