

LICENSEE EVENT REPORT

UPDATE REPORT
PREVIOUS REPORT DATE: 6/23/76

CONTROL BLE

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME 01 I L D R S 2														LICENSE NUMBER 00-000000-00										LICENSE TYPE 41111					EVENT TYPE 03														
7	8	9	14	15	25	26	30	31	32	CATEGORY 01 CONT										REPORT TYPE L		REPORT SOURCE L		DOCKET NUMBER 050-0237										EVENT DATE 052776					REPORT DATE 070676				
7	8	9	57	58	59	60	61	68	69	74	75	80	7	8	9	57	58	59	60	61	68	69	74	75	80																		

EVENT DESCRIPTION

02 DURING ROUTINE SURVEILLANCE TESTING, REACTOR PROTECTION SYSTEM 600 PSI BYPASS PRESSURE																																																																															
03 SWITCH PS 2-263-51A WAS FOUND TO ACTUATE AT 600 PSI (TECH SPEC LIMIT: LESS THAN 600 PSI)																																																																															
04 SYSTEM OPERABILITY WAS NOT AFFECTED BECAUSE REDUNDANT PRESSURE SWITCH 2-263-51B																																																																															
05 OPERATED PROPERLY. PS 2-263-51A WAS ADJUSTED TO THE PROPER SETPOINT. REPORTABLE																																																																															
06 SETPOINT DRIFT ON SIMILAR SWITCHES HAS OCCURRED SEVERAL TIMES. (50-237/1976-37)																																																																															

SYSTEM CODE 07 I A										CAUSE CODE E		COMPONENT CODE I N S T R U										PRIME COMPONENT SUPPLIER N		COMPONENT MANUFACTURER M 2 2 5										VIOLATION Y	
7	8	9	10	11	12	17	43	44	47	48																									

CAUSE DESCRIPTION

08 THE EVENT WAS CAUSED BY INSTRUMENT SETPOINT DRIFT. PS 2-263-51A WILL BE																																																																															
09 INSPECTED ON A MONTHLY BASIS UNTIL IT CONSISTENTLY EXHIBITS WHAT THE STATION																																																																															
10 CONSIDERS TO BE ACCEPTABLY CONSERVATIVE DRIFT.																																																																															

FACILITY STATUS 11 C										% POWER 025										OTHER STATUS NA										METHOD OF DISCOVERY B										DISCOVERY DESCRIPTION NA									
7	8	9	10	12	13	44	45	46	80																																								

FORM OF ACTIVITY RELEASED 12 Z										CONTENT OF RELEASE Z										AMOUNT OF ACTIVITY NA										LOCATION OF RELEASE NA									
7	8	9	10	11	44	45	80																																

PERSONNEL EXPOSURES

NUMBER 13 000										TYPE Z		DESCRIPTION NA									
7	8	9	11	12	13	80															

PERSONNEL INJURIES

NUMBER 14 000										DESCRIPTION NA									
7	8	9	11	12	80														

OFFSITE CONSEQUENCES

15 NA																																																																															
7	8	9	80																																																																												

LOSS OR DAMAGE TO FACILITY

TYPE 16 Z										DESCRIPTION NA									
7	8	9	10	80															

PUBLICITY

17 NA																																																																															
7	8	9	80																																																																												

ADDITIONAL FACTORS

18																																																																															
7	8	9	80																																																																												

19																																																																															
7	8	9	80																																																																												

NAME: G. REIMERS

PHONE: EXT. 265



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

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BBS Ltr. #76-511

July 6, 1976

Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Enclosed please find an update report to Reportable Occurrence Report number 50-237/1976-37. This report is being submitted to your office in accordance with the Dresden Nuclear Power Station Technical Specifications, Section 6.6.B.

B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:jo

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

7047

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