

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 2 3 4 5 6

PLEASE PRINT ALL REQUIRED INFORMATION

LICENSEE NAME 01 I L D R S 2														LICENSE NUMBER 15 0 - 0 0 0 0 0 - 0 0										LICENSE TYPE 26 4 1 1 1 1					EVENT TYPE 31 0 1	
7 8 9		14		15		25		26		30		31		32																
01 CONT		CATEGORY 57 58		REPORT TYPE 59 T		REPORT SOURCE 60 L		DOCKET NUMBER 61 0 5 0 - 0 2 3 7		EVENT DATE 69 1 2 2 8 7 6		REPORT DATE 75 0 3 1 0 7 7		80																
7 8 9		57 58		59		60		61		69		74		80																

EVENT DESCRIPTION

02 During start-up operations, control rod J-2 was withdrawn one notch. The reactivity														80	
03 inserted by this notch withdrawal resulted in a transient power increase. A														80	
04 reactor scram on IRM high flux prevented the transient from attaining a measur-														80	
05 able stable period; however, period instrumentation indicated transient readings														80	
06 of approximately 5 seconds. (Continued)														80	
7 8 9														80	

SYSTEM CODE 07 Z Z		CAUSE CODE 11 F		COMPONENT CODE 12 Z Z Z Z Z Z				PRIME COMPONENT SUPPLIER 43 Z		COMPONENT MANUFACTURER 44 Z 9 9 9				VIOLATION 48 Y	
7 8 9 10		11		12				43		44				48	

CAUSE DESCRIPTION

08 A combination of existing conditions such as the control rod pattern, moderator														80	
09 density, and local xenon concentration apparently resulted in an unexpectedly														80	
10 high reactivity worth for the notch. The resulting transient increase tripped														80	
7 8 9														80	

FACILITY STATUS 11 C		% POWER 10 0 0 0		OTHER STATUS 13 NA		METHOD OF DISCOVERY 44 A		DISCOVERY DESCRIPTION 46 Reactor Short Period, Scram					
7 8 9		10		13		44		46					
FORM OF ACTIVITY RELEASED 12 Z		CONTENT OF RELEASE 10 Z		AMOUNT OF ACTIVITY 11 NA				LOCATION OF RELEASE 45 NA				80	
7 8 9		10		11				45				80	

PERSONNEL EXPOSURES

NUMBER 13 0 0 0		TYPE 12 Z		DESCRIPTION 13 NA		80	
7 8 9		11		12		13	

PERSONNEL INJURIES

NUMBER 14 0 0 0		DESCRIPTION 12 NA		80	
7 8 9		11		12	

OFFSITE CONSEQUENCES

15 NA		80	
7 8 9		80	

LOSS OR DAMAGE TO FACILITY

TYPE 16 Z		DESCRIPTION 16 NA		80	
7 8 9		A10		80	

PUBLICITY

17 NA		80	
7 8 9		80	

ADDITIONAL FACTORS

18 NA		80	
7 8 9		80	

19		80	
7 8 9		80	

8304050033 770310
PDR ADDCK 05000237
S PDR

NAME: James G. Toscas

PHONE: Ext. 464

EVENT DESCRIPTION (Continued)

Analysis indicated that the reactivity insertion was substantially below the limit of $0.013\Delta K$ required by Tech Spec section 3.3.B.3.a., and that the equivalent stable period was 10 - 20 seconds. Furthermore, the withdrawal was performed in accordance with G.E. control rod withdrawal sequencing rules for reactor power levels below 20%. Since the reactor protection systems functioned as designed, this event was considered to be of little safety significance. This is the first reported occurrence of this nature at Dresden. (50-237/1976-74)

CAUSE DESCRIPTION (Continued)

the reactor before the operator could reinsert the control rod or adjust instrument ranges.

Following a normal scram recovery, start-up operations were resumed. Although the notch withdrawal of CRD J-2 was postponed until a later step in the control rod withdrawal sequence, all CRD movements remained in keeping with the G.E. start-up CRD withdrawal sequencing rules described above. Additionally, the nuclear engineers discussed the incident and developed methods of avoiding future potentially undesirable notch pulls under similar conditions.

As the result of an administrative oversight, this event was initially classified as a non-reportable occurrence. On 2/25/77, the station was notified that the occurrence was reportable because it represented a short-term reactivity increase corresponding to a reactor period of less than 5 seconds (Tech Spec section 6.6.B.1.d). As a result of this incident, administrative and supervisory personnel were reacquainted with this particular reporting requirement.



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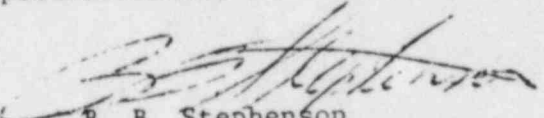
March 11, 1977



Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Enclosed please find Reportable Occurrence report number 50-237/1976-74.
This report is being submitted to your office in accordance with the Dresden
Nuclear Power Station Technical Specifications, Section 6.6.B.

IE FILE COPY


B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:jo

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

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