

January 31, 1974

Re: Indian Point Unit No. 2
AEC Docket No. 50-247
Facility Operating License
DPR-26
A.O.-4-2-6

Mr. James P. O'Reilly, Director
Regulatory Operations, Region I
U. S. Atomic Energy Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Dear Mr. O'Reilly,

On January 25, 1974 the reactor was brought critical preparatory to placing the plant back in service following completion of repairs associated with the November 13, 1973 feedwater line break incident. Criticality was achieved with the "C" bank of control rods 45 steps withdrawn from the core. The critical positions of the control rods were approximately 27 steps below the insertion limits shown on Figure 3.10-1 of the Technical Specifications. At the time of the occurrence the reactor coolant system pressure and temperature were 2235 psig. and 540°F respectively with all four reactor coolant pumps in service and the boron concentration was 1125 ppm boron. The 27 step differential represents a decrease in the required shutdown margin of about 0.26% reactivity. Minimum boron concentration required to maintain the required shutdown margin (Figure 3.10-3 of the Technical Specifications) is calculated to be 770 ppm. Therefore, the shutdown margin at the time of the occurrence was considerably more than that required by Figure 3.10-3.

We are investigating the causes of this occurrence and corrective action necessary to prevent recurrence. Mr. Anthony Pasano of your office was notified of this occurrence by telephone by Mr. John Makepeace on January 30, 1974.

Very truly yours,

Warren R. Cobean, Jr.
Warren R. Cobean, Jr., Manager
Nuclear Power Generation

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cc: Mr. John F. O'Leary

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