

LICENSEE EVENT REPORT (LER)

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|--|--|--------|-----|-----------|--|--------------|--|-------------------|--|---|--|-------|-------------------|--------|--|-----------------|--|--------------|--|--|--|--|-----|--|--|------|-----|--|--|---|--|--|--|--|--|--|-----------|--|--|--|--|--|--|--|--|--|--|--|--|
| FACILITY NAME (1) Dresden Nuclear Power Station, Unit 3 | | | | | | | | | | DOCKET NUMBER (2) 0 5 0 0 0 2 4 9 1 OF 0 2 | | | | | | | | | | PAGE (3) 1 OF 0 2 | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| TITLE (4) Reactor Scram | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| EVENT DATE (5) | | | | | | | | | | LER NUMBER (6) | | | | | | | | | | REPORT DATE (7) | | | | | | | | | | OTHER FACILITIES INVOLVED (8) | | | | | | | | | | | | | | | | | | | |
| MONTH | | | DAY | | | YEAR | | | | YEAR | | | SEQUENTIAL NUMBER | | | REVISION NUMBER | | | | MONTH | | | DAY | | | YEAR | | | | FACILITY NAMES | | | | | | | | | | DOCKET NUMBER(S) | | | | | | | | | |
| | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | N/A | | | | | | | | | | 0 5 0 0 0 | | | | | | | | | |
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| OPERATING MODE (9) N | | | | | | | | | | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11) | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| POWER LEVEL (10) 0 1 5 | | | | | | | | | | 20.402(b) | | | | | | | | | | 20.406(c) | | | | | | | | | | <input checked="" type="checkbox"/> 50.73(a)(2)(iv) | | | | | | | | | | 73.71(b) | | | | | | | | | |
| | | | | | | | | | | 20.406(a)(1)(i) | | | | | | | | | | 50.36(c)(1) | | | | | | | | | | 50.73(a)(2)(v) | | | | | | | | | | 73.71(c) | | | | | | | | | |
| | | | | | | | | | | 20.406(a)(1)(ii) | | | | | | | | | | 50.36(c)(2) | | | | | | | | | | 50.73(a)(2)(vii) | | | | | | | | | | OTHER (Specify in Abstract below and in Text, NRC Form 366A) | | | | | | | | | |
| | | | | | | | | | | 20.406(a)(1)(iii) | | | | | | | | | | 50.73(a)(2)(i) | | | | | | | | | | 50.73(a)(2)(viii)(A) | | | | | | | | | | | | | | | | | | | |
| | | | | | | | | | | 20.406(a)(1)(iv) | | | | | | | | | | 50.73(a)(2)(ii) | | | | | | | | | | 50.73(a)(2)(viii)(B) | | | | | | | | | | | | | | | | | | | |
| 20.406(a)(1)(v) | | | | | | | | | | 50.73(a)(2)(iii) | | | | | | | | | | 50.73(a)(2)(ix) | | | | | | | | | | 50.73(a)(2)(x) | | | | | | | | | | | | | | | | | | | |
| LICENSEE CONTACT FOR THIS LER (12) | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| NAME Gerald W. Bergan (X529) | | | | | | | | | | | | | | | | | | | | TELEPHONE NUMBER AREA CODE 8 1 1 5 9 1 4 1 2 1 - 1 2 1 9 1 2 1 0 | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| CAUSE | | SYSTEM | | COMPONENT | | MANUFACTURER | | REPORTABLE TO NRC | | | | CAUSE | | SYSTEM | | COMPONENT | | MANUFACTURER | | REPORTABLE TO NRC | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
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| SUPPLEMENTAL REPORT EXPECTED (14) | | | | | | | | | | | | | | | | | | | | EXPECTED SUBMISSION DATE (15) | | | | | | | | | | MONTH DAY YEAR | | | | | | | | | | | | | | | | | | | |
| YES (If yes, complete EXPECTED SUBMISSION DATE) | | | | | | | | | | | | | | | | | | | | <input checked="" type="checkbox"/> NO | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During a Reactor start-up subsequent to a refueling outage, with Reactor heat up and pressurization under way, Reactor water level decreased to the low level alarm point. With the low-flow feedwater regulating valve wide open in response to the low water level, the first Reactor feed pump was started. Excessive cold water entering the Reactor caused a high neutron flux Reactor scram.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31-85

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|--|--|-----------------|-------------------------------------|---------------------------------|----------|--|--|
| FACILITY NAME (1) Dresden Nuclear Power Station | DOCKET NUMBER (2) 0 5 0 0 0 2 4 9 | LER NUMBER (6) | | | PAGE (3) | | |
| | | YEAR 8 4 | SEQUENTIAL NUMBER — 0 0 1 | REVISION NUMBER — 0 0 | | | |

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Following a refueling outage, with Unit 3 Reactor in the startup mode, Reactor power had increased to the heating range with control rods being notched out to maintain proper heat up rate. A second Nuclear Station Operator (NSO) was assisting in maintaining the appropriate heat up rate by increasing the turbine pressure set. One condensate/booster pump was on and supplying coolant to the Reactor vessel. At 310 psig Reactor pressure and with two turbine steam bypass valves open, it was noticed by the NSO pulling control rods that the Reactor water level was decreasing and the low flow feedwater regulating valve was wide open. After verifying the condensate pump had not tripped, the NSO attempted to open a main feedwater isolation valve. While attempting to open the main feedwater isolation valve, water level continued to drop to the low level alarm point of 20 inches. The NSO then started a first Reactor feed pump to recover the loss of coolant inventory. This action introduced excessive cold water in the vessel via a wide open low flow feedwater regulating valve. The reactor scrammed on high neutron flux conditions at 15% APRM power while the reactor mode switch was in the startup position. All appropriate protective systems performed as designed.

Coolant loss was compounded by the fact that two bypass valves were open. Surveillances, DOS 250-5, Automatic Blowdown System at Low Pressure and Rated Pressure, and DOS 2300-3, HPCI System Pump Test, require some bypass steam flow as a prerequisite to avoid any Reactor pressure and flux transients. The two bypass valves remained open after testing, resulting in a drop in water level which the condensate/booster pump was unable to restore.

Future occurrences of this type will be minimized by reducing bypass steam flow, upon completion of the low pressure testing, prior to continuation of the Reactor heat up.



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

April 18, 1984

DJS Ltr. #84-392

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Licensee Event Report #84-001-0, Docket #050-249 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73(A)(2)(iv).

D. J. Scott
Station Superintendent
Dresden Nuclear Power Station

DJS/jmt

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III
File/NRC
File/Numerical

IE22
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