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James J. Fisicaro  
Director  
Nuclear Safety

May 19, 1995

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Mail Stop P1-37  
Washington, D.C. 20555

Subject: River Bend Station - Unit 1  
Docket No. 50-458  
License No. NPF-47  
Licensee Event Report 50-458/95-004-00  
File Nos. G9.5, G9.25.1.3

RBG-41532  
RBF1-95-0129

Gentlemen:

In accordance with 10CFR50.73, enclosed is the subject report. This is an interim report.

Sincerely,

JJF/RMM  
enclosure

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PDR ADOCK 05000458  
S PDR

JE22

Licensee Event Report 50-458/95-004-00  
May 19, 1995  
RBG-41532  
RBF1-95-0129  
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cc: U. S. Nuclear Regulatory Commission  
611 Ryan Plaza Drive, Suite 400  
Arlington, TX 76011

NRC Sr. Resident Inspector  
P. O. Box 1051  
St. Francisville, LA 70775

INPO Records Center  
700 Galleria Parkway  
Atlanta, GA 30339-3064

Mr. C. R. Oberg  
Public Utility Commission of Texas  
7800 Shoal Creek Blvd., Suite 400 North  
Austin, TX 78757

Louisiana Department of Environmental Quality  
Radiation Protection Division  
P.O. Box 82135  
Baton Rouge, LA 70884-2135  
ATTN: Administrator

NRC FORM 366 (5-92)		U.S. NUCLEAR REGULATORY COMMISSION			APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95					
<b>LICENSEE EVENT REPORT (LER)</b>								ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503		
FACILITY NAME (1) <b>River Bend Station</b>								DOCKET NUMBER (2) <b>05000-458</b>		PAGE (3) <b>1 of 3</b>
TITLE (4) <b>Auxiliary Building Door Determined Inoperable Due to Failure of Latching Mechanism</b>										
EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
04	19	95	95	004	00	05	19	95	N/A	05000
									FACILITY NAME	DOCKET NUMBER
									N/A	05000
OPERATING MODE (9)		1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more (11))							
POWER LEVEL (10)		100	20.402(b)			20.405(c)			50.73(a)(2)(iv)	73.71(b)
			20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)	73.71(c)
			20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)	OTHER
			20.405(a)(1)(iii)		X	50.73(a)(2)(i)			50.73(a)(2)(viii)(A)	(Specify in abstract below and in text, NRC Form 366A)
			20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)	
			20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)	
LICENSEE CONTACT FOR THIS LER (12)										
NAME <b>T. W. Gates, Supervisor - Nuclear Licensing</b>						TELEPHONE NUMBER (Include Area Code) <b>504-381-4866</b>				
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
SUPPLEMENTAL REPORT EXPECTED (14)						EXPECTED		MONTH	DAY	YEAR
X	YES (If yes, complete EXPECTED SUBMISSION DATE)		NO			SUBMISSION DATE (15)		07	13	95
<b>ABSTRACT</b> (Limit to 1400 spaces, i.e. approximately 15 single-spaced typewritten lines) (16)										
<p>On April 19, 1995 with the plant at 100 percent power (Operational Condition 1), while reviewing a maintenance work plan revision, Operations personnel concluded there was no latch mechanism available to maintain secondary containment door AB-70-04 closed. Technical Specification (TS) 4.6.5.1.b.2 requires that this door be maintained closed except during normal entry and exit and as such, this condition represents operation prohibited by the TS.</p> <p>The door was immediately returned to an acceptable configuration. A review of operating logs indicated that secondary containment was maintained during the period when the door was not held closed. An engineering evaluation indicates that, had the door been open, the safety function provided by secondary containment would have been fulfilled in the event of an accident. Therefore, this event was of no safety significance.</p> <p>The Root Cause for this event is currently being investigated by Entergy Operations, Inc. This determination and long term corrective actions will be provided in a supplemental report.</p>										

NRC FORM 366A (5-92)	U.S. NUCLEAR REGULATORY COMMISSION	APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95	
<p align="center"><b>LICENSEE EVENT REPORT (LER)</b> <b>TEXT CONTINUATION</b></p>		<p>ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.</p>	
		FACILITY NAME (1) <b>River Bend Station</b>	DOCKET NUMBER (2) <b>05000-458</b>

## REPORTED CONDITION

On April 19, 1995 with the plant at 100 percent power (Operational Condition 1), while reviewing a maintenance work plan revision which added the repair of previously unidentified failed door (\*DR\*) components, operations personnel concluded there was no latch mechanism available to maintain secondary containment door AB-70-04 closed. Technical Specification (TS) 4.6.5.1.b.2 requires that this door be maintained closed except during normal entry and exit. This condition represents a condition as plant operation prohibited by TS and is reportable pursuant 10CFR50.73(a)(2)(i)(B).

## INVESTIGATION

Auxiliary Building (\*NF\*) door AB-70-04 is a pressure tight secondary containment door. The door is operated by a centrally located handwheel that manipulates two independent latching units; upper and lower. Each latching unit consists of a main gear (\*GR\*) which manipulates three door latches (one on each vertical side and one on the horizontal) that are inserted into their corresponding guides to maintain the door closed.

Initial problems with door AB-70-04 occurred in November 1994 and were associated with the lower latching unit. The lower pivot bolt for the main gear had failed resulting in failure of the three associated latches. Problems experienced during the repair delayed resolution and restoration of the lower latching unit. As a result, the lower latching unit was removed and access through the door was maintained utilizing the upper latching unit which remained operable. Problems with the upper latching unit developed on April 13, 1995, at which time the door was removed from service.

On two occasions between the periods of April 13, 1995, through April 18, 1995, door AB-70-04 was found with the upper latches fully retracted leaving the door unlatched. On the first occasion, security officers discovered the condition and immediately inserted the latches securing the door. The second occurrence was identified by maintenance personnel on April 18, 1995, during a walkdown of the door related to the repair of the lower latching unit. At this time, maintenance determined that the upper latching unit had failed similarly to the lower latching unit. To prevent damage to the associated electrical components, maintenance personnel removed the non-functioning upper latching unit until the work package could be revised. The door remained closed but unlatched for approximately 24 hours until the operations department learned of the problem.

On April 19, during operations review of the revised work package, it was recognized that no latching mechanism existed to hold the door closed. Operations immediately declared the door inoperable and entered the action statement for Technical Specification 4.6.5.1 (Secondary Containment). The door's latching mechanisms were immediately installed and an operability evaluation was initiated to evaluate the condition.

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				PAGE (3) <b>3 OF 3</b>	

## ROOT CAUSE

Entergy Operations, Inc. (EOI) is evaluating the root cause of this event and will communicate the results in a supplemental report.

## CORRECTIVE ACTION

Secondary containment door (Auxiliary Building) AB-70-04 was immediately restored to an acceptable configuration. An engineering evaluation was performed which determined that the secondary containment function of the auxiliary building could have been fulfilled with the door in the as-found condition.

Long term corrective actions will be established upon completion of the root cause evaluation and will be documented in the supplemental report.

## SAFETY ASSESSMENT

The door was immediately returned to an acceptable configuration. A review of operating logs indicated that secondary containment was maintained during the period when the door was not held closed. Had the door opened, it is probable that the draw down capability of the standby gas treatment system (\*BH\*) (draw down to -0.25 inches of water within 13.5 seconds) established in TS 4.6.5.1.c.1 would have been exceeded. However, an engineering evaluation determined that with the expected increase in draw down time of about 0.4 seconds, the offsite dose limits would not have been exceeded even if the door had opened (or been open). This event therefore had no safety significance.

Note: Energy Industry Identification Codes are indicated in the text as (\*XX\*).