



GPU Nuclear Corporation
100 Interpace Parkway
Parsippany, New Jersey 07054
201 263-6500
TELEX 136-482
Writer's Direct Dial Number:

April 13, 1984

Mr. Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Crutchfield:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Safety Concerns Associated With Pipe Breaks
In BWR Scram System Piping (NUREG 0803)

By letter dated January 17, 1984, the NRC requested that GPU Nuclear Corporation provide plant specific responses conforming to the guidance in NUREG 0803.

It was our belief that the Systematic Evaluation Program (SEP) for Oyster Creek would address the concerns identified in NUREG 0803. The following items have been addressed by plant specific responses:

- Limitation of reactor coolant iodine concentration to standard Technical Specification values. Covered under SEP sections 4.36 and 4.37.
- Seismic design verification and as-built inspection of scram discharge volume piping and supports. Covered in response to IE Bulletin 79-14.

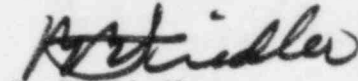
Presently the BWR Owner's Group (BWROG) is answering concerns associated with: 1) leak rate and stress calculations for the entire range of SDV header sizes (4" to 24") and, 2) leak detection capability in the scram discharge volume areas. These concerns are a direct consequence of the BWROG November 3, 1983 fracture mechanics evaluation submitted to the NRC and a February 23, 1984 NRC - BWROG/GE working meeting in Bethesda, Md.

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We are endorsing the BWROG position in this matter and consider that no further action is possible until the BWROG response is completed.

Very truly yours,



P. B. Fiedler
Vice President and Director
Oyster Creek

1r/0197e

cc: Administrator
Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pa. 19406

NRC Resident Inspector
Oyster Creek Nuclear Generating Station
Forked River, N. J. 08731