

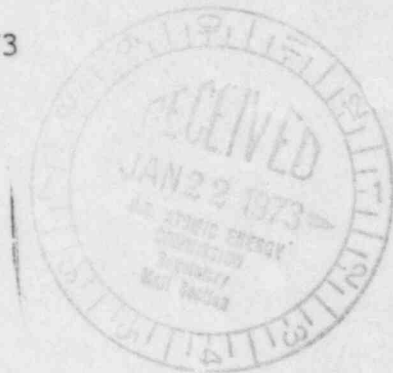
Jersey Central Power & Light Company



MADISON AVENUE AT PUNCH BOWL ROAD • MORRISTOWN, N. J. 07960 • 539-6111

January 16, 1973

Mr. Donald J. Skovholt
Assistant Director for Operating Reactors
Directorate of Licensing
United States Atomic Energy Commission
Washington, D. C. 20545



Dear Mr. Skovholt:

SUBJECT: OYSTER CREEK NUCLEAR GENERATING STATION
DOCKET 50-219
CORE SPRAY SUCTION HEADER
DYNAMIC ANALYSIS

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This letter is in response to your letter dated December 13, 1972 regarding the above subject. A comparison of the suction header piping and supports of the Quad Cities and Oyster Creek facilities has resulted in the conclusion that the failures experienced at Quad Cities 2 could not occur at Oyster Creek. Unlike the Quad Cities header support arrangement, the weight of the Oyster Creek ring header is supported by seventeen 14WFL30 vertical (pedestal-type) supports which are attached to the rigid concrete mat foundation. The only connections between the suppression chamber and the suction header are the three 20" x 20" x 16" inlet "T"'s and six pairs of hydraulic snubbers oriented in the horizontal plane. This arrangement is illustrated in figures 1, 3 and 4 of section 9 of Amendment 32 which was submitted on February 16, 1968.

JCA

On December 29, 1972, a turbine trip and reactor scram occurred; during the transient the pressure increased to above the setpoint of the electromatic relief valves. It is suspected that two to five relief valves actuated at that time and all but one resealed as designed. The remaining valve stayed open causing the plant to blowdown to the torus until the pressure in the reactor vessel was near atmospheric. This incident is the subject of an abnormal occurrence report being submitted to Mr. A. Ciambusso within the next few days. After this occurrence a thorough inspection of the core spray suction header support arrangement was made which revealed no abnormal conditions. Furthermore, there was no evidence of relative motion between the suction header and its vertical supports.

To be responsive to your request, however, we have contacted General Electric, the original designer of the system, and have learned that the information you have requested is not currently available. General Electric

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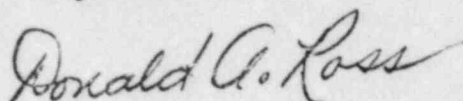
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felt that your request was not applicable to Oyster Creek considering our physical arrangement and the failures at Quad Cities 2 that prompted your request. General Electric informed us that tests and analyses were currently being conducted to determine the dynamic response resulting from a relief valve blowdown for plants similar to Quad Cities 2. As you know, General Electric is scheduled to issue a report of their findings in March 1973. After we have had the opportunity to study this report and discuss its applicability to Oyster Creek with General Electric, we will advise you of our plans concerning any further action we feel will be necessary to establish the adequacy of our present arrangement.

Very truly yours,



D. A. Ross, Manager
Nuclear Generating Stations

DAR:tm

cc: Mr. J. P. O'Reilly, Director
Directorate of Regulatory Operations, Region 1