



ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649

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AREA CODE 716 546-2700

March 20, 1984

Dr. Thomas E. Murley, Regional Administrator
U.S. Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

Subject: Potential Loss of Residual Heat Removal (RHR) Capability
R. E. Ginna Nuclear Power Plant, Unit No. 1
Docket No. 50-244

Dear Dr. Murley:

This is a preliminary notification of an event that occurred on March 7, 1984, that will be followed by a Licensee Event Report.

While at Cold Shutdown condition the following actions were in progress: draining the Reactor Coolant System (RCS) to the CVCS hold up tanks, and preparing to shift from draining the RCS via the Reactor Coolant Drain Tank Pump to the low pressure purification pump. During this process Valves MOV 851 A and B (Containment Sump B suction to RHR) were opened prior to shutting valve MOV 850A (Downstream of MOV 851A and Upstream of RCDT pump suction).

The Operator's first action was to turn the switch for MOV-850A to the "Closed" position. Approximately 30 seconds later the Sump B level alarm was actuated and the 8" level indicator lights were actuated. The operator then noticed that the RCS loop level monitor indicated zero. The running RHR pump was then stopped. To this point the RHR flow had remained at a steady 900 GPM. After MOV-850A was shut, the RHR pump was restarted and 900 GPM flow indicated. Approximately one minute later MOV-856 (Refueling Water Storage Tank to RHR Pump Suction and to RCS Hotleg Loop A) was opened to restore RCS loop level, then closed. The final pressurizer level indication was 150". Delaying the opening of MOV-856 until after the RHR pump was restarted and indicating steady flow, showed that at no time was RHR inoperable.

The RHR temperature prior to the event was approximately 100°F. When the pump was stopped the RHR temperature indicated approximately 115°F. When MOV 856 was opened the temperature decreased to approximately 60°F. When RHR flow was reestablished the temperature returned to 100°F.

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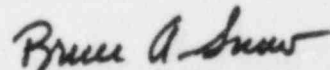
Both CAM's (continuous air monitors) in the Containment Vessel Basement were alarming on High Gaseous Activity. R-12 (Containment Vent Gaseous Monitor) indication in the Control Room did not increase significantly above the pre-event reading. Containment evacuation alarm was sounded and personnel inside containment evacuated.

One hour or four hour notification of the event to the NRC was evaluated and it was determined that none of the criteria established in the Ginna Procedure O-9.3, which covers all 10 CFR 50.72 reporting criteria were applicable.

Subsequent investigation revealed that a total of 31 inches of water was drained into the Sump B (Sump B discrete light indications are at 8 inches and 78 inches). This was verified by actual measurement. This volume of approximately 2000 gallons is consistent with the calculation of RCS loop level change from 20 inches to zero inches. The evaluation further showed that after opening valves MOV-851A and B the RHR pumps were still operable and operating. The event that could have caused loss of RHR capability was the closing of MOV-850A with essentially zero inches loop level indication. Had the RHR pumps operated in this mode for greater than or equal to one minute without MOV-856 open they could have become air bound and hence inoperable. It should be noted that the travel time for 851A, 851B, 850A, and 856 is approximately two minutes and calculations show that it would take less than 30 seconds to drain the volume of water from RCS (20" loop level to zero) to Sump B.

Further evaluation and discussions revealed that this event is reportable under 10 CFR 50.73 (a) (v) which thus implies it is reportable within four hours of occurrence, in that the event alone could have prevented the fulfillment of the safety function of system needed to remove residual heat, without operator action.

Very truly yours,



Bruce A. Snow
Plant Superintendent