

QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

MARCH 1984

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for Unit Two.

This report was compiled by Becky Brown and Alex Misak, telephone number 309-654-2241, extensions 127 and 194.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

Unit One began the month derated to 710 MWe due to End of Cycle fuel depletion. These deratings continued until 0113 hours on March 6 when the unit was shutdown for End of Cycle Seven Refueling and maintenance. In addition, the unit dropped load to 700 MWe at 0020 hours on March 4 for weekly Turbine tests. At 0105 hours, the testing was completed, and a normal load increase was initiated.

B. Unit Two

March 1-5: Unit Two began the month increasing load 5 MWe/hour to full power following a unit shutdown to replace the motor on the 2B Recirculation Pump Discharge Valve. This load increase was terminated at 790 MWe on March 2 because of high Turbine vibration. The unit maintained this load until 0340 hours on March 3 when the unit dropped load to 650 MWe due to Condensate Demineralizer problems. At 0700 hours, a normal load increase was initiated. This load increase was terminated at 0050 hours on March 4 due to high Turbine vibration. The unit maintained a load of 790 MWe until 0130 hours on March 5 when the unit dropped load to 650 MWe due to Condensate Demineralizer problems. At 0315 hours the unit began increasing load, and the increase continued until 1530 hours when the unit held load at 790 MWe due to high Turbine vibration.

March 6-11: At 0305 hours on March 6, the unit dropped load to 650 MWe due to Condensate Demineralizer problems. At 0550 hours the unit began a normal load increase to full power. At 0315 hours on March 7 the unit dropped load to 750 MWe for Main Steam Isolation Valve testing. At 0355 hours the testing was completed, and the unit increased load until 0420 hours when the increase was terminated because of high Turbine vibration. At 0508 hours, the unit dropped load to 600 MWe due to high Turbine vibration. At 0800 hours the unit began increasing load to full power. At 2345 hours, on March 10, the unit dropped load to 700 MWe for weekly Turbine tests and Control Rod Pattern adjustments. At 0220 hours the unit began a normal load increase.

B. Unit Two (Continued)

March 12-23: At 0655 hours on March 17, the unit dropped load to 660 MWe due to a failed Feedwater Regulating Valve. At 0000 hours on March 18, the valve was repaired and a normal load increase to full power was initiated.

March 24-31: At 2110 hours on March 24, the unit dropped load to 300 MWe as part of testing to collect data on single Recirculation pump operation and to define the flow control line. At 0330 hours on March 25, the testing was completed and the unit began a normal load increase to full power.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY
RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

On February 14, 1984, the NRC issued Amendment 81 to License DPR-30. This Amendment establishes limiting conditions for operation, setpoints for overvoltage, undervoltage, and underfrequency protective instrumentation, and surveillance requirements for the Reactor Protection Bus Power Monitoring System.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Unit One and Unit Two during the reporting period. This summary includes the following headings; Work Request Numbers, LER Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT ONE MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q33089		RHR Suppression Pool Cooling Valve 1-1091-36B	Leads were broken at the limit contact block.	The valve would not operate; the unit was shutdown.	All of the lugs on the limit contact block connected to field wiring were replaced.
Q33386	84-2	A0 1-1601-23	Wear.	Contributed total LLRT failure.	The packing was adjusted.
Q33387	84-2	A0 1-1601-24	Wear.	Contributed total LLRT failure.	The plate opposite the operator had a gasket installed.

UNIT TWO MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q29139		2A RHR Service Water Pump	Cavitation.	The pump makes a lot of noise and the casing is erroded.	The problem is being studied. Meanwhile, the casing was replaced.
Q32602		IRM Channel 15 216 Dual Trip Module	The dual trip module was bad.	The IRM gave a constant inoperable signal.	The 216 Dual trip module was replaced with a spare.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1. and 6.6.B.2. of the Technical Specifications.

<u>UNIT ONE</u>		
<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
84-1	3-5-84	Electromatic Failed to Open
84-2	3-7-84	Leak Rate From all Valves and Penetrations 293.75 SCFH
84-3	3-6-84	Spurious Water Level Scram
84-4	3-16-84	MSIV Failed Local Leak Rate Test

UNIT TWO

There were no Licensee Event Reports, for Unit Two, for the reporting period.

V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

OPERATING DATA REPORT

DOCKET NO. 50-254

UNIT ONE

DATE April 6

COMPLETED BY Alex L. Misak

TELEPHONE 309-654-2241x194

OPERATING STATUS

0000 030184

1. Reporting period: 2400 033184 Gross hours in reporting period: 744

2. Currently authorized power level (MWt): 2511 Max. Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted (if any) (MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr. to Date	Cumulative
5. Number of hours reactor was critical	<u>122.1</u>	<u>1562.1</u>	<u>85117.7</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>3421.9</u>
7. Hours generator on line	<u>121.2</u>	<u>1561.2</u>	<u>81909.1</u>
8. Unit reserve shutdown hours.	<u>0.0</u>	<u>0.0</u>	<u>907.2</u>
9. Gross thermal energy generated (MWH)	<u>252848</u>	<u>3659732</u>	<u>168766438</u>
10. Gross electrical energy generated (MWH)	<u>83636</u>	<u>1213148</u>	<u>54471764</u>
11. Net electrical energy generated (MWH)	<u>79579</u>	<u>1152950</u>	<u>50758917</u>
12. Reactor service factor	<u>16.4</u>	<u>71.5</u>	<u>81.7</u>
13. Reactor availability factor	<u>16.4</u>	<u>71.5</u>	<u>84.9</u>
14. Unit service factor	<u>16.3</u>	<u>71.5</u>	<u>78.6</u>
15. Unit availability factor	<u>16.3</u>	<u>71.5</u>	<u>79.5</u>
16. Unit capacity factor (Using MDC)	<u>13.9</u>	<u>68.6</u>	<u>63.3</u>
17. Unit capacity factor (Using Des. MWe)	<u>13.6</u>	<u>66.9</u>	<u>61.7</u>
18. Unit forced outage rate	<u>0.0</u>	<u>0.0</u>	<u>6.1</u>

19. Shutdowns scheduled over next 6 months (Type, Date, and Duration of each):

20. If shutdown at end of report period, estimated date of start up 7-30-84

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

*UNOFFICIAL COMPANY NUMBERS ARE USED IN THIS REPORT

OPERATING DATA REPORT

DOCKET NO. 50-265

UNIT TWO

DATE April 6

COMPLETED BY Alex L. Misak

TELEPHONE 309-654-2241x194

OPERATING STATUS

0000 030184

1. Reporting period: 2400 033184 Gross hours in reporting period: 744

2. Currently authorized power level (MWt): 2511 Max. Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted (if any) (MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr. to Date	Cumulative
5. Number of hours reactor was critical	<u>744.0</u>	<u>989.5</u>	<u>78907.0</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>2985.8</u>
7. Hours generator on line	<u>744.0</u>	<u>904.3</u>	<u>76114.1</u>
8. Unit reserve shutdown hours.	<u>0.0</u>	<u>0.0</u>	<u>702.9</u>
9. Gross thermal energy generated (MWH)	<u>1783500</u>	<u>1977283</u>	<u>157359371</u>
10. Gross electrical energy generated (MWH)	<u>583515</u>	<u>641733</u>	<u>50077513</u>
11. Net electrical energy generated (MWH)	<u>554974</u>	<u>608312</u>	<u>46942372</u>
12. Reactor service factor	<u>100.0</u>	<u>45.3</u>	<u>76.4</u>
13. Reactor availability factor	<u>100.0</u>	<u>45.3</u>	<u>79.3</u>
14. Unit service factor	<u>100.0</u>	<u>41.4</u>	<u>73.7</u>
15. Unit availability factor	<u>100.0</u>	<u>41.4</u>	<u>74.4</u>
16. Unit capacity factor (Using MDC)	<u>97.0</u>	<u>36.2</u>	<u>59.1</u>
17. Unit capacity factor (Using Des. MWe)	<u>94.5</u>	<u>35.3</u>	<u>57.6</u>
18. Unit forced outage rate	<u>0.0</u>	<u>10.6</u>	<u>8.7</u>

19. Shutdowns scheduled over next 6 months (Type, Date, and Duration of each):

20. If shutdown at end of report period, estimated date of startup NA

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

*UNOFFICIAL COMPANY NUMBERS ARE USED IN THIS REPORT

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-254

UNIT ONE

DATE April 6

COMPLETED BY Alex L. Misak

TELEPHONE 309-654-2241x194

MONTH March 1984

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>654.3</u>
2.	<u>697.3</u>
3.	<u>692.8</u>
4.	<u>709.6</u>
5.	<u>559.8</u>
6.	<u>-6.6</u>
7.	<u>-10.7</u>
8.	<u>-9.8</u>
9.	<u>-10.2</u>
10.	<u>-9.8</u>
11.	<u>-9.8</u>
12.	<u>-10.2</u>
13.	<u>-8.2</u>
14.	<u>-7.9</u>
15.	<u>-6.5</u>
16.	<u>-5.7</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>-5.8</u>
18.	<u>-5.4</u>
19.	<u>-5.3</u>
20.	<u>-5.3</u>
21.	<u>-5.7</u>
22.	<u>-5.6</u>
23.	<u>-5.5</u>
24.	<u>-5.2</u>
25.	<u>-4.1</u>
26.	<u>-4.2</u>
27.	<u>-4.2</u>
28.	<u>-4.0</u>
29.	<u>-4.2</u>
30.	<u>-3.3</u>
31.	<u>-3.2</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-265

UNIT TWO

DATE April 6

COMPLETED BY Alex L. Misak

TELEPHONE 309-654-2241x194

MONTH March 1984

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>689.8</u>
2.	<u>745.7</u>
3.	<u>708.8</u>
4.	<u>738.3</u>
5.	<u>729.9</u>
6.	<u>757.6</u>
7.	<u>677.8</u>
8.	<u>744.4</u>
9.	<u>780.8</u>
10.	<u>763.2</u>
11.	<u>745.3</u>
12.	<u>769.1</u>
13.	<u>775.7</u>
14.	<u>785.5</u>
15.	<u>770.8</u>
16.	<u>780.8</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>679.4</u>
18.	<u>697.1</u>
19.	<u>781.9</u>
20.	<u>786.3</u>
21.	<u>773.0</u>
22.	<u>780.9</u>
23.	<u>783.7</u>
24.	<u>733.3</u>
25.	<u>600.7</u>
26.	<u>748.1</u>
27.	<u>783.5</u>
28.	<u>778.3</u>
29.	<u>781.4</u>
30.	<u>781.6</u>
31.	<u>779.0</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

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APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13

Revision 6

August 1982

DOCKET NO. 050-254

UNIT NAME Quad-Cities Unit One

COMPLETED BY Alex Misak

DATE April 9, 1984

REPORT MONTH MARCH 1984

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
84-12	840304	S	0.0	B	5		HA	XXXXXX	Reduced load to perform weekly Turbine tests
84-13	840305	F	0.0	B	5	84-1	SF	VALVEX	Reduced load for Drywell entry to inspect 'IE' Electromatic Relief Valve
84-14	840306	S	622.8	C	1	84-1	RC	FUELXX	Unit One Shutdown for End of Cycle Seven Refueling and Maintenance

APPROVED

AUG 16 1982

ID/5A

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-265UNIT NAME Quad-Cities Unit TwoCOMPLETED BY Alex MisakDATE April 9, 1984REPORT MONTH MARCH 1984TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
84-5	840303	F	0.0	B	5		HG	DEMINX	Reduced load due to Condensate Demineralizer problems
84-6	840305	F	0.0	B	5		HG	DEMINX	Reduced load due to Condensate Demineralizer problems
84-7	840306	F	0.0	B	5		HG	DEMINX	Reduced load due to Condensate Demineralizer problems
84-8	840307	S	0.0	B	5		CD	VALVEX	Reduced load for weekly MSIV testing
84-9	840307	F	0.0	B	5		HA	TURBIN	Reduced load due to high Turbine vibration
84-10	840310	S	0.0	B	5		KB	CONKOD	Reduced load for Control Rod Pattern adjustments
84-11	840317	F	0.0	B	5		CH	VALVEX	Reduced load due to failed Feedwater Regulating Valve
84-12	840324	S	0.0	B	5		CB	ZZZZZZ	Reduced load for Flow Control Line determination and single Recirculation Loop operation data collection.

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VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission.

A. Main Steam Relief Valve Operations

There were no Main Steam Relief Valve Operations for the reporting period.

B. Control Rod Drive Scram Timing Data For Units One and Two

There was no Control Rod Drive Scram Timing Data for Units One and Two for the reporting period.

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities, and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QUAD-CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 1
March 1978

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1. Unit: Q1 Reload: 7 Cycle: 8
Refueling Outage
Currently in Progress
2. Scheduled date for next refueling shutdown: 5-21-84
3. Scheduled date for restart following refueling: 5-21-84
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment: Yes. Preparatory Technical Specification changes have been submitted to include MAPLHGR curve for one of the reload fuel types and extending MAPLHGR curve for BLTA to 45,000 MWD/t.
5. Scheduled date(s) for submitting proposed licensing action and supporting information:
Technical Specification change has been submitted February 21, 1984.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
- 1) All new fuel assemblies will be GE7B-type (barrier clad, extended exposure design).
 - 2) A generic methodology was used for the analysis of the Control Rod Drop Accident and Rod Withdrawal Error events.
 - 3) Four Barrier Lead Test Assemblies will be re-inserted to gather information on the effects of extended exposures.
7. The number of fuel assemblies.
- a. Number of assemblies in core: 0
 - b. Number of assemblies in spent fuel pool: 2454
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
- a. Licensed storage capacity for spent fuel: 3657
 - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2003

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APR 20 1978

Q. C. O. S. R.

QUAD-CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 1
March 1978

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1. Unit: Q2 Reload: 7 Cycle: 8
2. Scheduled date for next refueling shutdown: 3-18-85
3. Scheduled date for restart following refueling: 5-26-85
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment:

Not as yet determined.
5. Scheduled date(s) for submitting proposed licensing action and supporting information:

January 18, 1985, if licensing action required.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
 - 1) All new fuel assemblies will be GE7B-type (barrier clad, extended exposure design).
 - 2) A generic methodology was used for the analysis of the Control Rod Drop Accident and Rod Withdrawal Error events.
7. The number of fuel assemblies.
 - a. Number of assemblies in core: 724
 - b. Number of assemblies in spent fuel pool: 412
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
 - a. Licensed storage capacity for spent fuel: 3897
 - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2003

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Q. C. O. S. R.

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	-	Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	-	American National Standards Institute
APRM	-	Average Power Range Monitor
ATWS	-	Anticipated Transient Without Scram
BWR	-	Boiling Water Reactor
CRD	-	Control Rod Drive
EHC	-	Electro-Hydraulic Control System
EOF	-	Emergency Operations Facility
GSEP	-	Generating Stations Emergency Plan
HEPA	-	High-Efficiency Particulate Filter
HPCI	-	High Pressure Coolant Injection System
HRSS	-	High Radiation Sampling System
IPCIRT	-	Integrated Primary Containment Leak Rate Test
IRM	-	Intermediate Range Monitor
ISI	-	Inservice Inspection
LER	-	Licensee Event Report
LIRT	-	Local Leak Rate Test
LPCI	-	Low Pressure Coolant Injection Mode of RHRS
LPRM	-	Local Power Range Monitor
MAPLHGR	-	Maximum Average Planar Linear Heat Generation Rate
MCPR	-	Minimum Critical Power Ratio
MFLCPR	-	Maximum Fraction Limiting Critical Power Ratio
MPC	-	Maximum Permissible Concentration
MSIV	-	Main Steam Isolation Valve
NIOSH	-	National Institute for Occupational Safety and Health
PCI	-	Primary Containment Isolation
PCOMR	-	Preconditioning Interim Operating Management Recommendations
RBCCW	-	Reactor Building Closed Cooling Water System
RBM	-	Rod Block Monitor
RCIC	-	Reactor Core Isolation Cooling System
RHRS	-	Residual Heat Removal System
RPS	-	Reactor Protection System
RWM	-	Rod Worth Minimizer
SBGTS	-	Standby Gas Treatment System
SBLC	-	Standby Liquid Control
SDC	-	Shutdown Cooling Mode of RHRS
SDV	-	Scram Discharge Volume
SRM	-	Source Range Monitor
TBCCW	-	Turbine Building Closed Cooling Water System
TIP	-	Traversing Incore Probe
TSC	-	Technical Support Center



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Telephone 309/654-2241

NJK-84-118

April 9, 1984

Director, Office of Inspection & Enforcement
United States Nuclear Regulatory Commission
Washington, D. C. 20555
Attention: Document Control Desk

Gentlemen:

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of March 1984.

Very truly yours,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

N. J. Kalivianakis
Station Superintendent

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Enclosure

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