

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
 UNIT Fort Calhoun Statio
 DATE April 13, 1984
 COMPLETED BY T. P. Matthews
 TELEPHONE (402) 536-4733

MONTH March, 1984

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	341.8
2	232.4
3	0.0
4	0.0
5	0.0
6	0.0
7	0.0
8	0.0
9	0.0
10	0.0
11	0.0
12	0.0
13	0.0
14	0.0
15	0.0
16	0.0

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	0.0
18	0.0
19	0.0
20	0.0
21	0.0
22	0.0
23	0.0
24	0.0
25	0.0
26	0.0
27	0.0
28	0.0
29	0.0
30	0.0
31	0.0

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

B404190163 B40331
 PDR ADOCK 05000285
 R PDR

OPERATING DATA REPORT

DOCKET NO. 50-285
 DATE April 13, 1984
 COMPLETED BY T. P. Matthews
 TELEPHONE (402) 536-4733

OPERATING STATUS

1. Unit Name: Fort Calhoun Station
2. Reporting Period: March, 1984
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 501
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 461
7. Maximum Dependable Capacity (Net MWe): 438
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

Notes

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: None

	This Month	Yr.-to-Date *	Cumulative
11. Hours In Reporting Period	744.0	2,184.0	92,186.0
12. Number Of Hours Reactor Was Critical	50.2	1,490.2	71,384.1
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.0
14. Hours Generator On-Line	49.5	1,489.5	70,892.0
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	49,026.4	2,152,796.9	88,912,510.6
17. Gross Electrical Energy Generated (MWH)	14,816.0	690,258.0	29,007,827.0
18. Net Electrical Energy Generated (MWH)	13,781.8	656,536.5	27,736,405.2
19. Unit Service Factor	6.7	68.2	76.9
20. Unit Availability Factor	6.7	68.2	76.9
21. Unit Capacity Factor (Using MDC Net)	4.2	68.6	65.6
22. Unit Capacity Factor (Using DER Net)	3.9	62.9	63.3
23. Unit Forced Outage Rate	0.0	0.0	3.5
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: May 1, 1984

26. Units In Test Status (Prior to Commercial Operation): N/A Forecast Achieved

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH March, 1984

DOCKET NO. 50-285
 UNIT NAME Fort Calhoun Station
 DATE April 13, 1984
 COMPLETED BY T. P. Matthews
 TELEPHONE (402) 536-4733

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
84-01	840303	S	694	C	4	N/A	XX	XXXXXX	1984 refueling outage commenced March 3, 1984.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending March 1984.

1. Scheduled date for next refueling shutdown. September 1985
2. Scheduled date for restart following refueling. November 1985
3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes
 - a. If answer is yes, what, in general, will these be?

Technical Specification change

- b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. _____
 - c. If no such review has taken place, when is it scheduled? _____
4. Scheduled date(s) for submitting proposed licensing action and support information. August 1985
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
6. The number of fuel assemblies:

a) in the core	<u>133</u>	assemblies
b) in the spent fuel pool	<u>305</u>	"
c) spent fuel pool storage capacity	<u>729</u>	"
d) planned spent fuel pool storage capacity	<u>*</u>	"

*may be increased via fuel pin consolation
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1996

Prepared by J. K. Gasser

Date April 1, 1984

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

March, 1984
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station went off line March 3, 1984, for a scheduled maintenance and refueling outage.

Several major jobs which were completed or are in progress during March include:

1. All four reactor coolant pumps had seals and case-to-cover gaskets replaced.
2. Eddy current testing was performed on both steam generators.
3. Both steam generators have been sludge lanced.
4. The reactor vessel head has been removed and preparations are under way to begin fuel movement.
5. The five year inspection has been performed on the main generator.
6. Inspection and repair on the turbine continues along with installation of the new high pressure turbine rotor.

No safety valve or PORV challenges occurred.

A. PERFORMANCE CHARACTERISTICS

None

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

None

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

<u>Procedure</u>	<u>Description</u>
SP-NFR-1	Fuel Receipt Procedure. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because new fuel was received and inspected in accordance with approved procedures and the limitations of the Station operating license.
SP-PORV-2	PORV Setpoints. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because it was completed as planned using normal plant procedures.
SF-FIL-2	Inspection and Pressure Drop Measurement for Technical Support Center Cooling and Filtering Unit. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because it did not involve safety related equipment.

System Acceptance Committee Packages for March, 1984: *

<u>Package</u>	<u>Description/Analysis</u>
EEAR FC-83-100	Chemistry Hot Lab Vacuum Pump Installation. This modification is not safety related; therefore, has no adverse effect on the safety analysis.
EEAR FC-78-20	Generator Trip with Generator Breaker Mod. Open. This modification makes it so that the generator breakers will not close out of synchronization and cause a large transient. This modification has no adverse effect on the safety analysis.
EEAR FC-79-134	Piping from Room 59 to Switchgear Room for Containment Leak Rate Test. This modification prevents having to run instrument piping through fire doors when performing the 60 psi containment leak rate test. This modification has no adverse effect on the safety analysis.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL
(continued)

System Acceptance Committee Packages for March, 1984: (continued)

<u>Procedure</u>	<u>Description</u>
EEAR FC-79-92	Secondary Plant Sampling System. This modification is not safety related; therefore, has no adverse effect on the safety analysis.
EEAR FC-79-87	Installation of Additional Gai-tronics Stations. This modification is not safety related; therefore, has no adverse effect on the safety analysis.
EEAR FC-80-02	Load Lifting Eye. This modification is not safety related; therefore, has no adverse effect on the safety analysis.
EEAR FC-79-38	Containment Ventilation Modification for Low Volume Flows. This modification improves the control over the containment purges for low volume flows. This modification has no adverse effect on the safety analysis.
EEAR FC-80-133	Modification of Health Physics Work Area. This modification involved relocation of a detector. This detector is to be verified operable by CP-FDZ-6 after it is reinstalled. This change will not effect its operability and the requirements of Technical Specification 2.19.1 will be met. This modification has no adverse effect on the safety analysis.
EEAR FC-82-82	Lift Rig for Containment Recirculation Strainer Baskets. This modification provided for the fabrication of lift rig for containment recirculation strainer baskets. This device is used to lift the strainer baskets when the plant is in a shutdown condition. This modification has no adverse effect on the safety analysis.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL
(continued)

System Acceptance Committee Packages for March, 1984: (continued)

<u>Procedure</u>	<u>Description</u>
EEAR FC-80-6	Mast Rotate/Spreader Interlock. This modification provided for the installation of extra safety interlock on PAR refueling machine. This modification has no adverse effect on the safety analysis.
EEAR FC-82-17	Replacement of DG1 and DG2 Pressure Switches. This modification will enhance the safety of the system by providing better quality equipment. This modification has no adverse effect on the safety analysis.

E. RESULTS OF LEAK RATE TESTS

The Fort Calhoun Station is currently performing B and C penetration tests. A report will be sent out at the end of the refueling outage.

F. CHANGES IN PLANT OPERATING STAFF

Mr. Mark Gardner started March 27, 1984, as an Auxiliary Operator - Nuclear.

G. TRAINING

Training in March was directed toward outage requirements in the areas of General Employee Training, crane training, system hydro test training and job briefings for ALARA requirements.

One new operator completed five weeks of initial plant training.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION
AUTHORIZATION PURSUANT TO 10CFR50.59

None

Monthly Operations Report
March, 1984
Page Five

II. MAINTENANCE (Significant Safety Related)

None

L.T. Kusek
for W. Gary Gates
Manager
Fort Calhoun Station

Omaha Public Power District
1623 Harney Omaha, Nebraska 68102
402/536-4000

April 13, 1984
LIC-84-111

Mr. Richard C. DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Reference: Docket No. 50-285

Dear Mr. DeYoung:

Please find enclosed ten (10) copies of the March Monthly
Operating Report for the Fort Calhoun Station Unit No. 1.

Sincerely,



W. C. Jones
Division Manager
Production Operations

WCJ/TPM:jmm

Enclosures

cc: NRC Regional Office
Office of Management & Program Analysis (2)
Mr. R. R. Mills - Combustion Engineering
Mr. T. F. Polk - Westinghouse
Nuclear Safety Analysis Center
INPO Records Center
NRC File

IE24
1/1