Omaha Public Power District
444 South 16th Street Mall
Omaha, Nebraska 68102-2247
402/636-2000

May 15, 1995
LTC-95-0105

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-137
Washington, DC 20555

Reference: Docket No. 50-285

SUBJECT: April 1995 Monthly Operating Report (MOR)

Enclosed please find the April 1995 MOR for Fort Calhoun Station (FCS) Unit No. 1 as required by FCS Technical Specification 5.9.1.

If you should have any questions, please contact me.

Sincerely,

T. L. Patterson
Division Manager
Nuclear Operations

TLP/dll

Enclosures

c: Winston & Strawn
L. J. Callan, NRC Regional Administrator, Region IV
T. Y. Liu, NRC Acting Project Manager
W. C. Walker, NRC Senior Resident Inspector
R. T. Pearce, Combustion Engineering
R. J. Simon, Westinghouse
INPO Records Center

JE24

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

APRIL 1995
Monthly Operating Report

1. OPERATIONS SUMMARY

The Fort Calhoun Station (FCS) 1995 Refueling Outage was completed during the month of April. The plant performed the cold hydrostatic (hydro) pressure test on April 4, 1995, and then continued with plant heatup and pressurization. The Reactor Coolant System (RCS) hot hydro was completed satisfactorily on April 8th, after a return to cold shutdown to repair a leak on HCV-151, one of the two Power Operated Relief Valves (PORV) block valves. Low Power Physics Testing was completed by April 13th and the generator was synchronized to the grid on Friday, April 14, 1995, ending the 53-day refueling outage. On April 21, 1995, power ascension to a nominal 100% level was completed. Power was reduced to 95% on April 28th, to complete Moderator Temperature Coefficient (MTC) testing, with power returning to 100% on May 1.

Work was completed on the modification to the Control Room air conditioners to address a design basis Component Cooling Water (CCW) temperature issue.

Inspection No. 95-04, the NRC Resident Monthly Inspection, was completed during this reporting period. Licensee Event Report (LER) 95-001, "Time Delay Relays for Offsite Power Low Signal Found Out-of-Tolerance" was submitted during this reporting period.

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

During the month of April, no PORV or Primary System Safety Valve challenges or failures occurred.

3. RESULTS OF LEAK RATE TESTS

Trending of the RCS leak rate commenced with the return to normal power operation following the completion of the 1995 Refueling Outage. Transient conditions during plant startup prevented meaningful trending of the daily leak rate results. With the return to steady state conditions towards the end of the month, the leak rate was steady between 0.100 and 0.200 gpm with no degrading trends noted.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Amendment No.	Description
None	

5. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF APRIL 1995

- Continued troubleshooting the high vibration problem on Auxiliary Feedwater Pump FW-54.

- Repaired and modified Raw Water Pumps AC-10B and AC-10C Sparging System following plugging of Sparging System Strainer CW-19 and subsequent sanding in of AC-10B.

- Investigated the spurious tripping of 480 VAC Motor Control Center 4A1 feeder circuit breaker following a ground on 480 VAC bus 1B4A. This circuit breaker contains General Electric RMS-9 solid state trip units which have been the cause of previous spurious circuit breaker trips. As a precautionary measure, the trip unit was subsequently replaced.

- Repaired PORV block valve HCV-151 seal leak.

- Replaced clutch assemblies on Control Element Drive Mechanisms (CEDMs) RC-10-03, -14, -26, -38 and -39. Also replaced seal housing assemblies on CEDMs RC-10-18, -26, -32 and -35.

- Replaced various Agastat relays as an equipment reliability issue.

- Replaced four Rosemount Model 1154 transmitters which were identified as being a 10 CFR Part 21 concern.

6. OPERATING DATA REPORT

Attachment I

7. AVERAGE DAILY UNIT POWER LEVEL

Attachment II

8. UNIT SHUTDOWNS AND POWER REDUCTIONS

Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

Attachment IV

ATTACHMENT I
OPERATING DATA REPORT

DOCKET NO. 50-285
UNIT FORT CALHOUN STATION
DATE MAY 04,1995
COMPLETED BY D. L. LIPPY
TELEPHONE (402) 533-6843

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
2. Reporting Period: APRIL 1995

NOTES

3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Elec. Rating (Net MWe): 478
6. Max. Dep. Capacity (Gross MWe): 502
7. Max. Dep. Capacity (Net MWe): 478

8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:
N/A

9. Power Level to which restricted, if any (Net MWe): N/A

10. Reasons for restrictions, if any:
N/A

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period.....	719.0	2879.0	189313.0
12. Number of Hours Reactor was Critical	436.6	1653.3	148071.2
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	389.7	1602.0	146376.3
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	484384.7	2255337.1	194404037.6
17. Gross Elec. Energy Generated (MWH)..	161136.0	762732.0	64167614.2
18. Net Elec. Energy Generated (MWH)....	152917.0	727024.4	61218816.7
19. Unit Service Factor.....	54.2	55.6	77.3
20. Unit Availability Factor.....	54.2	55.6	77.3
21. Unit Capacity Factor (using MDC Net)	44.5	52.8	70.0
22. Unit Capacity Factor (using DER Net)	44.5	52.8	68.3
23. Unit Forced Outage Rate.....	.0	.0	4.0

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
NONE

25. If shut down at end of report period, estimated date of startup: _____

26. Units in test status (prior to comm. oper.):

		Forecast	Achieved
INITIAL CRITICALITY			
INITIAL ELECTRICITY	N/A		
COMMERCIAL OPERATION			

ATTACHMENT II
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	MAY 04, 1995
COMPLETED BY	D. L. LIPPY
TELEPHONE	(402) 533-6843

MONTH APRIL 1995

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	146
2	0	18	363
3	0	19	407
4	0	20	452
5	0	21	478
6	0	22	486
7	0	23	487
8	0	24	487
9	0	25	486
10	0	26	487
11	0	27	487
12	0	28	486
13	0	29	460
14	0	30	461
15	87	31	N/A
16	112		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ATTACHMENT III
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
UNIT NAME Fort Calhoun St.
DATE May 5, 1995
COMPLETED BY D. L. Lippy
TELEPHONE (402) 533-6843

REPORT MONTH April 1995

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
95-02	950220	S	329.3	C	1	N/A	ZZ	ZZZZZZ	On February 20, 1995 Fort Calhoun Station (FCS) commenced its 15th refueling outage. The generator was synchronized to the grid on April 14, ending the FCS 1995 Refueling Outage.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit F - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit H - Same Source

(9/77)

ATTACHMENT IV
REFUELING INFORMATION
Fort Calhoun Station - Unit No. 1

Report for the month ending April 30, 1995

1. Scheduled date for next refueling shutdown. — September 21, 1996

2. Scheduled date for restart following refueling. — November 2, 1996

3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment? — No

   a. If answer is yes, what, in general, will these be? — N/A

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. — No

   c. If no such review has taken place, when is it scheduled? — Prior to November 1996

4. Scheduled date(s) for submitting proposed licensing action and support information. — No submittal planned

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. — * / **

6. The number of fuel assemblies:
   a) in the core — 133 Assemblies
   b) in the spent fuel pool — 618 Assemblies
   c) spent fuel pool storage capacity — 1083 Assemblies

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. — 2007 Outage

* OPPD utilized methodology employing the CASMO-3/SIMULATE-3codes for reactor physics related analyses for Cycle 16. This methodology was approved by the NRC in December 1994.

** Seven fuel assemblies found with suspected fuel pin failures. Fuel assembly reconstitution effort, which included high magnification visual examination of the individual rods, concluded that defects were a result of grid-to-rod fretting. Confirmed failed pins were replaced with stainless steel rods for Cycle 16 operation.

Prepared by Kevin Hall — Date 5-5-95