

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

MARCH 1984

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

Document 0043r

8404180003 840331  
PDR ADOCK 05000373  
R PDR

## TABLE OF CONTENTS

- I. INTRODUCTION
- II. MONTHLY REPORT FOR UNIT ONE
  - A. Summary of Operating Experience
  - B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE
    - 1. Amendments to Facility License or Technical Specifications
    - 2. Facility or Procedure Changes Requiring NRC Approval
    - 3. Tests and Experiments Requiring NRC Approval
    - 4. Corrective Maintenance of Safety Related Equipment
  - C. LICENSEE EVENT REPORTS
  - D. DATA TABULATIONS
    - 1. Operating Data Report
    - 2. Average Daily Unit Power Level
    - 3. Unit Shutdowns and Power Reductions
  - E. UNIQUE REPORTING REQUIREMENTS
    - 1. Main Steam Relief Valve Operations
    - 2. ECCS System Outages
    - 3. Off-Site Dose Calculation Manual Changes
    - 4. Major Changes to Radioactive Waste Treatment System

## I. INTRODUCTION

The LaSalle Nuclear Power Station is a Two Unit Facility Located in Marseilles, Illinois. Each Unit is a Boiling Water Reactor with a designed electrical output of 1078 MWe net. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. Unit One is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. The unit commenced commercial generation of power on January 1, 1984. Unit Two is subject to license number NFP-18, issued on December 16, 1983. The date of initial criticality was March 10, 1984. The Unit is expected to commence commercial generation of power in August, '84.

This report was compiled by Randy S. Dus, telephone number (815)357-6761, extension 324.

II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

MARCH 1-5 The Unit started the reporting period in cold shutdown due to repairs on the extraction steam line expansion bellows and the condenser boot seal.

MARCH 6-18 The reactor went critical at 2240 hours on March 6. At 1235 hours on March 7 the main generator was synchronized to the grid. At 1508 hours on March 7 reactor power was at 25%. At 2130 hours on March 7 a Rx shutdown was commenced to analyze stress loading of temporary ventilation ductwork in drywell. At 0050 hours on March 8 the turbine generator was tripped. At 0640 on March 8, the Rx was shutdown. The reactor was critical for 32 hours and 0 minutes.

MARCH 19-31 The reactor went critical at 0315 hours on March 19. At 2335 hours on March 19, the main generator was synchronized to the Grid. Reactor power was raised to 23% at 1730 hours on March 20. At 2200 hours on March 20, reactor power was raised to 44%. At 0700 hours on March 21, reactor power was increased to 64%. Reactor power was reduced to 39% at 1830 hours on March 22 for a drywell entry to investigate the source of water in the "B" Rx recirc motor casing. At 0700 hours on March 24, reactor power was increased to 67%. Reactor power was decreased to 47% at 1645 hours on March 27 due to a feedwater heater string isolation. At 1150 hours on March 28, reactor power was reduced to 23% due to additional feedwater heater problems. At 1720 hours on March 29, commenced pulling rods to increase power. At 0700 hours on March 30, reactor power was at 70%. The reactor was critical for 308 hours and 45 minutes.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to Facility License or Technical Specifications.

Amendment No. 16- This amendment revised the electrical power systems Technical Specification 3.8.1.1. The Amendment changes the LaSalle Unit One Technical Specification Requirements for fast starts on the Diesel Generators, consistent with the provisions of the LaSalle Unit Two Technical Specifications. This involves the reduction in the number of required fast, cold start surveillances.

2. Facility or Procedure Changes Requiring NRC Approval.

There were no facility or procedure changes requiring NRC approval during the reporting period.

3. Tests and Experiments Requiring NRC Approval.

There were no tests or experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

The following tables present a summary of safety-related maintenance completed on Unit One during the reported period. The headings indicated in this summary include: Work Request Numbers, LER Numbers, Component Name, Cause of Malfunctions, Results and Effects on Safe Operation, and Corrective Action.

## ATTACHMENT A

LTP-300-7  
Revision 3  
March 1, 1983  
5

CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L24025		Bottom Head Drain Valve	Valve Seat Eroded	Additional Drywell Equipment Drain Sump Input	Line Temporarily capped until valves can be repaired.
L31534		Control Rod	Rod uncoupled when fully withdrawn.	Loss of Rod Control	Changed out Control Rod
L32576		D/G Daytank fill valve	Valve Does not open fully	Unable to fill day tank completely	Reset limits on fill valve.
L32968	84-012-00	H <sub>2</sub> Recombiner Isolation valves	Valve failed local leak rate test	Potential degradation of primary containment integrity	Repaired valve
L33098	84-012-00	RX Bldg Equip drains isola- tion valves	Valve Failed local leak rate test	Potential degradation of primary containment integrity	Repaired valve
L33680		Div I Post LOCA O <sub>2</sub> Rec	Erratic Indication Less than 20%	No effect on operation. Div II still operable.	Replaced faulty Resistor and Recalibrated.
L33778		RHR HX Steam Inlet Stop valve	Valve has dual indication when closed	valve position unknown	Limit switch reset
L33802		A IRM	BAD connection under vessel	IRM indicates upscale	Cleaned connection under vessel.



CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L33871		1RE024	Open limit switch contacts not picking up	Will not allow RX Bldg Equip drain sump to start	Adjusted limit switch actuator and bracket.
L34054	84-017-00	Ammonia/ Chlorine Detector	Defective Detectors	Will not isolate control room supply dampers	Repaired Detectors and recalibrated
L34155		RWCU Inboard Isolation valve	Packing leak	Leakage into drywell sump	Valve Repacked
L34185		"A" RHR HX bypass valve	Overload contacts open	Loss of position indication	Replaced thermal overloads.
L34399		Stack Gas WRGM recorder	Purge Gas Bottle pressure low causing faulty reading	Control Room indication is high	Replaced purge Gas Bottle
L34402		Rx Level Indicating Switch	Switch Intermittently contacts	Inaccurate Rx Level Indication	Replaced microswitch
L34556		Rx Mode switch	Contacts for run mode input to Rx manual control system not picked up.	Causes "Rod out Block"	Cleaned circuit board ground contacts
L34929		VR Rad Monitor	Defective Monitor does not alarm or indicate	None. Redundant channels still operable	Replaced faulty relay board
L34947		Ammonia Detector	Detector wired incorrectly	Would not isolate supply dampers on high ammonia signal	Rewired and verified proper operation

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, March 1 through March 31, 1984. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in section 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
84-008-00	2/18/84	IRM Cable Bumped/Non-coincident Reactor Scram
84-009-00	2/19/84	Upscale spike on "A" IRM/Non-coincident Scram
84-010-00	2/19/84	RPS Actuation from "A" IRM/Non-coincident Scram
84-011-00	2/13/84	Rx scram due to condenser boot Seal failure
84-012-00	2/14/84	Leak rate limit exceeded
84-013-00	2/14/84	Main steam line high flow isolation
84-014-00	3/1/84	Procedure error in LES-RI-01
84-015-00	2/27/84	Group II and Group IV isolation
84-016-00	3/7/84	RHR B & C injection valves Rx Pressure Interlock
84-017-00	3/8/84	Failure of control room ventilation ammonia/chlorine detection system



D. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

## OPERATING DATA REPORT

DOCKET NO. 050-373  
UNIT LaSalle One  
DATE April 15, 1984  
COMPLETED BY Randy S. Dus  
TELEPHONE (815)357-6761

## OPERATING STATUS

1. REPORTING PERIOD: March 1984 GROSS HOURS IN REPORTING PERIOD: 744  
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3323 MAX DEPEND CAPACITY  
(MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078  
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A  
4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	<u>340.8</u>	<u>1158.5</u>	<u>1158.5</u>
6. REACTOR RESERVE SHUTDOWN HOURS	<u>403.2</u>	<u>992.6</u>	<u>992.6</u>
7. HOURS GENERATOR ON LINE	<u>300.7</u>	<u>1035.6</u>	<u>1035.6</u>
8. UNIT RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>1.0</u>	<u>1.0</u>
9. GROSS THERMAL ENERGY GENERATED (MWH)	<u>578129</u>	<u>2381801</u>	<u>2381801</u>
10. GROSS ELEC. ENERGY GENERATED (MWH)	<u>182356</u>	<u>759266</u>	<u>759266</u>
11. NET ELEC. ENERGY GENERATED (MWH)	<u>165230</u>	<u>704862</u>	<u>704862</u>
12. REACTOR SERVICE FACTOR	<u>45.8%</u>	<u>53.0%</u>	<u>53.0%</u>
13. REACTOR AVAILABILITY FACTOR	<u>100%</u>	<u>98.5%</u>	<u>98.5%</u>
14. UNIT SERVICE FACTOR	<u>40.4%</u>	<u>47.4%</u>	<u>47.4%</u>
15. UNIT AVAILABILITY FACTOR	<u>40.4%</u>	<u>47.5%</u>	<u>47.5%</u>
16. UNIT CAPACITY FACTOR (USING MDC)	<u>21.4%</u>	<u>31.2%</u>	<u>31.2%</u>
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	<u>20.6%</u>	<u>29.9%</u>	<u>29.9%</u>
18. UNIT FORCED OUTAGE RATE	<u>48.8%</u>	<u>46.1%</u>	<u>46.1%</u>
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)			
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:		<u>N/A</u>	
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):			
	FORECAST	ACHIEVED	
INITIAL CRITICALITY	<u>          </u>	<u>6/21/82</u>	
INITIAL ELECTRICITY	<u>          </u>	<u>9/04/82</u>	
COMMERCIAL OPERATION	<u>          </u>	<u>1/1/84</u>	

LTP-300-7  
Revision 3  
March 1, 1983  
6

ATTACHMENT B  
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373  
UNIT: LASALLE ONE  
DATE: APRIL 15, 1984  
COMPLETED BY: RANDY S. DUS  
TELEPHONE: (815) 357-6761

MONTH March 1984

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1. _____ 0.0	17. _____ 0.0
2. _____ 0.0	18. _____ 0.0
3. _____ 0.0	19. _____ 0.0
4. _____ 0.0	20. _____ 206
5. _____ 0.0	21. _____ 682
6. _____ 0.0	22. _____ 639
7. _____ 70	23. _____ 456
8. _____ 0.0	24. _____ 708
9. _____ 0.0	25. _____ 736
10. _____ 0.0	26. _____ 846
11. _____ 0.0	27. _____ 764
12. _____ 0.0	28. _____ 321
13. _____ 0.0	29. _____ 204
14. _____ 0.0	30. _____ 759
15. _____ 0.0	31. _____ 804
16. _____ 0.0	

INSTRUCTIONS

On this form list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line.) In such cases the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

LTP-300-7  
Revision 3  
March 1, 1983  
9 (Final)

ATTACHMENT E  
UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH MARCH 1984

DOCKET NO. 050-373  
UNIT NAME LaSalle One  
DATE April 15, 1984  
COMPLETED BY Randy S. Dus  
TELEPHONE (815)357-6761

NO.	DATE	TYPE	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED S: SCHEDULED				
6	2/13/84	S	156.5	B	4	Condenser Boot Seal and extraction steam expansion joint repaired
7	3/8/84	F	286.7	F	1	Temporary drywell vent- ilation ductwork evaluated for loading on containment structural members. Analysis O.K. No corrective action taken. Remained shutdown to perform electrical cable butt splices inspection required by NRC.

E. UNIQUE REPORTING REQUIREMENTS

1. Main Steam Relief Valve Operations for Unit 1.

There were no relief valve operations for Unit One for this reporting period.

## 2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
1-227-84	"D" RHR Service Water Pump	Megger Motor
1-230-84	1A D/G	Lubrication
1-265-84	"C" RHR Service Water Pump	Remove Breaker to use on Unit Two.
1-275-84	"A" RHR HX Bypass Valve	Inspect Limits and Torques
1-276-84	"A" RHR HX	Repair conductivity Cell Valve
1-334-84	HPCS Water Leg Pump	Replace

## 3. Off-Site Dose Calculation Manual

There were no changes to the Off-Site Dose Calculations Manual during this reporting period.

## 4. Radioactive Waste Treatment System

There were no changes to the Radioactive Waste Treatment System during this reporting period.



LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

MARCH 1984

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

## TABLE OF CONTENTS

- I. INTRODUCTION
- II. MONTHLY REPORT FOR UNIT TWO
  - A. Summary of Operating Experience
  - B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE
    - 1. Amendments to Facility License or Technical Specifications
    - 2. Facility or Procedure Changes Requiring NRC Approval
    - 3. Tests and Experiments Requiring NRC Approval
    - 4. Corrective Maintenance of Safety Related Equipment
  - C. LICENSEE EVENT REPORTS
  - D. DATA TABULATIONS
    - 1. Operating Data Report
    - 2. Average Daily Unit Power Level
    - 3. Unit Shutdowns and Power Reductions
  - E. UNIQUE REPORTING REQUIREMENTS
    - 1. Main Steam Relief Valve Operations
    - 2. ECCS System Outages
    - 3. Off-Site Dose Calculation Manual Changes
    - 4. Major Changes to Radioactive Waste Treatment System

II MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

MARCH 1-9 The Unit started the reporting period in a shutdown condition. Initial criticality has not yet been achieved

MARCH 10 The Unit achieved initial criticality at 0808 hours on March 10. The reactor was shutdown at 1650 hours on March 10 per STP 6.2, Seq. B-2. The reactor was critical for 8 hours and 42 minutes.

MARCH 10-25 The reactor went critical at 2027 hours on March 10. At 2200 hours on March 10, the reactor was shutdown per STP 6.2, Seq A-2. The reactor was critical for 1 hour and 33 minutes.

MARCH 26-29 The reactor went critical at 0330 hours on March 26. At 1715 hours on March 29, the reactor was manually scrammed to repair a leak on the MDRFP discharge piping low point drain. The reactor was critical for 85 hours and 45 minutes.

MARCH 30-31 The reactor went critical at 1925 hours on March 30. The reactor was critical for 28 hours and 35 minutes.

E. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to Facility License or Technical Specifications.

Amendment No. 1 - This amendment revised paragraphs 2.c.(1) and 2.c.(15) of operating license No. NPF-18. Paragraph 2.c.(1) authorizes the operation of the Unit at full power level (3323 MWT). Paragraph 2.c.(15) outlines the required changes to be made to the fire protection system to effectively reduce system frictional losses.

2. Facility or Procedure Changes Requiring NRC Approval.

There were no facility or procedure changes requiring NRC approval during the reporting period.

3. Tests and Experiments Requiring NRC Approval.

There were no tests or experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

The following tables present a summary of safety-related maintenance completed on Unit One during the reported period. The headings indicated in this summary include: Work Request Numbers, LER Numbers, Component Name, Cause of Malfunctions, Results and Effects on Safe Operation, and Corrective Action.

## ATTACHMENT A

CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENTLTP-300-7  
Revision 3  
March 1, 1983  
5

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L33274		SBLC Pump Suction line	Heat Tracing insulation broken off.	Solidification at low temps	Replaced insulation
L33855		"B" RHR Service Water Pump	Outboard bearing damaged	Potential pump failure	Replaced bearing
L33861		Div. II post LOCA sample Pump	Pump failure	None. Div. I monitor still operable	Rebuilt pump
L33988		"B" IRM	Defective Detector	Causes spikes and half scrams	Replaced Detector
L34037		Reactor Recirc Pump Flow Transmitter	Transmitter Defective	Recirc Flow unknown	Replaced faulty circuit board and recalibrated
L34110		"D" IRM	Does not respond correctly	Causes upscale spikes	Repaired connection from range switch
L34158		RHR shutdown cooling isolation valve	Failed local leak rate test	Potential degradation of containment integrity	Machined seat to remove clearance.
L34198		Control wiring Butt Splices	Inadequate Butt Splices	Potential wire breaks	Repair Butt Splices per specification

## ATTACHMENT A (Cont'd)

CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENTLTP-300-7  
Revision 3  
March 1, 1983  
5

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L34263		Inboard feed- water check valve	Failed local leak rate test	Potential degradation of containment integrity	Machined hinge pins to reduce side clearance
L34320		RHR "A" water leg Pump Disch	Check valve does not seat	Permits reverse flow thru water leg pump from RHR system.	Removed and cleaned seat and disc.
L34554		Suppression pool wide range level transmitter	Transmitter does not respond properly/ sensing line clogged	Gives wrong suppression pool level	Cleaned sensing line and recalibrated
L34706		Mechanical Snubber	Snubber movement restricted due to grating interference	Will prevent piping move- ment on heatup	Notched grating to provide clearance.
L34980		"A" VP Fan	Breaker keeps tripping after 20 minutes.	Potential high drywell temps.	Replaced breaker



C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, occurring during the reporting period, March 1 through March 31, 1984. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in section 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
84-001-00	1/5/84	"E" IRM/Reactor Scram
84-003-00	2/2/84	Reactor Scram
84-003-00	2/2/84	Failure of ACB 2432 to trip
84-004-00	2/12/84	Div. II PCIS Isolation
84-005-00	2/15/84	HPCS Pump Feeder Breaker
84-006-00	2/23/84	RWCU High $\Delta T$ Isolation
84-007-00	2/18/84	RWCU High $\Delta T$ Isolation
84-008-00	2/29/84	Group I Isolation Reset Malfunction
84-009-00	3/8/84	RHR SDC Isolation

#### D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

## OPERATING DATA REPORT

DOCKET NO. 050-374  
UNIT LaSalle Two  
DATE April 15, 1984  
COMPLETED BY Randy S. Gas  
TELEPHONE (815)357-6761

## OPERATING STATUS

1. REPORTING PERIOD: March 1984 GROSS HOURS IN REPORTING PERIOD: 744  
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3323 MAX DEPEND CAPACITY  
(MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078  
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A  
4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	<u>124.5</u>	<u>124.5</u>	<u>124.5</u>
6. REACTOR RESERVE SHUTDOWN HOURS	<u>395.3</u>	<u>395.3</u>	<u>395.3</u>
7. HOURS GENERATOR ON LINE	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
8. UNIT RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
9. GROSS THERMAL ENERGY GENERATED (MWH)	<u>8278</u>	<u>8278</u>	<u>8278</u>
10. GROSS ELEC. ENERGY GENERATED (MWH)	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
11. NET ELEC. ENERGY GENERATED (MWH)	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
12. REACTOR SERVICE FACTOR	<u>N/A</u>	<u>N/A</u>	<u>N/A</u>
13. REACTOR AVAILABILITY FACTOR	<u>N/A</u>	<u>N/A</u>	<u>N/A</u>
14. UNIT SERVICE FACTOR	<u>N/A</u>	<u>N/A</u>	<u>N/A</u>
15. UNIT AVAILABILITY FACTOR	<u>N/A</u>	<u>N/A</u>	<u>N/A</u>
16. UNIT CAPACITY FACTOR (USING MDC)	<u>N/A</u>	<u>N/A</u>	<u>N/A</u>
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	<u>N/A</u>	<u>N/A</u>	<u>N/A</u>
18. UNIT FORCED OUTAGE RATE	<u>N/A</u>	<u>N/A</u>	<u>N/A</u>
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)			
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:		<u>N/A</u>	
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):			
	FORECAST	ACHIEVED	
INITIAL CRITICALITY		<u>3/10/84</u>	
INITIAL ELECTRICITY	<u>April 84</u>		
COMMERCIAL OPERATION	<u>Aug. 84</u>		

LTP-300-7  
Revision 3  
March 1, 1983  
6

ATTACHMENT B  
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374  
UNIT: LASALLE TWO  
DATE: APRIL 15, 1984  
COMPLETED BY: RANDY S. DUS  
TELEPHONE: (815) 357-6761

MONTH March 1984

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1. _____ 0.0	17. _____ 0.0
2. _____ 0.0	18. _____ 0.0
3. _____ 0.0	19. _____ 0.0
4. _____ 0.0	20. _____ 0.0
5. _____ 0.0	21. _____ 0.0
6. _____ 0.0	22. _____ 0.0
7. _____ 0.0	23. _____ 0.0
8. _____ 0.0	24. _____ 0.0
9. _____ 0.0	25. _____ 0.0
10. _____ 0.0	26. _____ 0.0
11. _____ 0.0	27. _____ 0.0
12. _____ 0.0	28. _____ 0.0
13. _____ 0.0	29. _____ 0.0
14. _____ 0.0	30. _____ 0.0
15. _____ 0.0	31. _____ 0.0
16. _____ 0.0	

INSTRUCTIONS

On this form list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line.) In such cases the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

LTP-300-7  
Revision 3  
March 1, 1983  
9 (Final)

ATTACHMENT E  
UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH MARCH 1984  
CONTINUED

DOCKET NO. 050-374  
UNIT NAME LaSalle Two  
DATE April 15, 1984  
COMPLETED BY Randy S. Dus  
TELEPHONE (815)357-6761

---

NO.	DATE	TYPE	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED S: SCHEDULED				

---

NONE

## E. UNIQUE REPORTING REQUIREMENTS

### 1. Main Steam Relief Valve Operations for Unit 2.

Relief valve operations during the reporting period are summarized in the following Table. The table included information as to which relief valve was actuated, how it was activated and the circumstances resulting in its actuation.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO &amp; TYPE ACTUATIONS</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
3/27/84	2B21-F013A	1 Manual	250 PSIG	STP-26
3/27/84	2B21-F013B	1 Manual	250 PSIG	STP-26
3/27/84	2B21-F013C	8 Manual	250 PSIG	STP-26
3/27/84	2B21-F013D	8 Manual	250 PSIG	STP-26
3/27/84	2B21-F013E	8 Manual	250 PSIG	STP-26
3/27/84	2B21-F013F	1 Manual	250 PSIG	STP-26
3/27/84	2B21-F013G	1 Manual	250 PSIG	STP-26
3/27/84	2B21-F013H	2 Manual	250 PSIG	STP-26
3/27/84	2B21-F013J	1 Manual	250 PSIG	STP-26
3/27/84	2B21-F013K	3 Manual	250 PSIG	STP-26
3/27/84	2B21-F013L	1 Manual	250 PSIG	STP-26
3/27/84	2B21-F013M	2 Manual	250 PSIG	STP-26
3/27/84	2B21-F013N	1 Manual	250 PSIG	STP-26
3/27/84	2B21-F013P	3 Manual	250 PSIG	STP-26
3/27/84	2B21-F013R	8 Manual	250 PSIG	STP-26
3/27/84	2B21-F013S	8 Manual	250 PSIG	STP-26
3/27/84	2B21-F013U	8 Manual	250 PSIG	STP-26
3/27/84	2B21-F013V	13 Manual	250 PSIG	STP-26



## 2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-357-84	"C" RHR Service Water Pump	Test Overload
2-359-84	2A D/G	Lubrication
2-364-84	A RHR Service Water Pump	Bearing Repair
2-365-84	B RHR Service Water Pump	Bearing/Shaft Inspection
2-368-84	HPCS D/G and Bus 243	Inspect AC/DC Bus
2-387-84	"C" & "D" RHR Service Water Pump	LES-GM-120
2-429-84	"A" RHR Water Leg Fill Valve	Repair Valve
2-465-84	RHR Service Water Strainer	Repair Strainer
2-468-84	"B" RHR Service Water Pump	Lubrication

## 3. Off-Site Dose Calculation Manual

There were no changes to the Off-Site Dose Calculations Manual during this reporting period.

## 4. Radioactive Waste Treatment System

There were no changes to the Radioactive Waste Treatment System during this reporting period.



**Commonwealth Edison**  
LaSalle County Nuclear Station  
Rural Route #1, Box 220  
Marseilles, Illinois 61341  
Telephone 815/357-6761

April 15, 1984

Director, Office of Management Information  
and Program Control  
United States Nuclear Regulatory Commission  
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for the period covering March 1 through March 31, 1984.

Very truly yours,

G. J. Diederich  
Superintendent  
LaSalle County Station

GJD/RSD/bej

Enclosure

xc: J. G. Keppler, NRC, Region III  
NRC Resident Inspector LaSalle  
Gary Wright, Ill. Dept. of Nuclear Safety  
D. P. Galle, CECO  
D. L. Farrar, CECO  
INPO Records Center  
Ron A. Johnson, PIP Coordinator SNED  
W. R. Jackson, GE Resident

IE24  
1/1