

PUBLIC SERVICE COMPANY OF COLORADO
FORT ST. VRAIN NUCLEAR GENERATING STATION

MONTHLY OPERATIONS REPORT

NO. 122

March, 1984

FORM 260 22 0218

8404180001 840331
PDR ADCK 05000267
R PDR

This report contains the highlights of the Fort St. Vrain, Unit No. 1, activities operated under the provisions of the Nuclear Regulatory Commission Operating License DPR-34. This report is for the month of March, 1984.

1.0 NARRATIVE SUMMARY OF OPERATING EXPERIENCE AND MAJOR SAFETY RELATED MAINTENANCE

Refueling was completed early in March.

Operations primary tasks during March were performing tests on the 480 V buses to assure operability and hanging or removing clearances on the 480V buses.

Loop 1 and 1B helium circulator maintained core cooling throughout the reporting period. "1A" helium circulator change out was completed this month, and the circulator will be placed in service early in April. The reactor vessel was maintained subatmospheric until the last week of March when reactor pressure was increased to 150 PSIG in preparation for epoxy injection on selected core support floor cooling tubes.

The orifices are being stroked daily to maintain operability. Difficulty continues with two orifice valves. All control rod position indications have been calibrated for data logger input.

The second twelve week session of non-licensed operator system training has been completed and the third session is in progress. Annual requalification training classes are also in progress.

The last week of March, during the performance of the surveillance for the inspection of the tendons, several tendon wires were found broken due to corrosion. There is no indication of tendon inoperability, but additional inspections and tests are underway to verify this. Evaluation of this situation continues.

2.0 SINGLE RELEASES OF RADIOACTIVITY OR RADIATION EXPOSURE IN EXCESS OF 10% OF THE ALLOWABLE ANNUAL VALUE

None.

3.0 INDICATION OF FAILED FUEL RESULTING FROM IRRADIATED FUEL EXAMINATIONS

None.

4.0 MONTHLY OPERATING DATA REPORT

Attached.

OPERATING DATA REPORT

DOCKET NO. 50-267
DATE April 6, 1984
COMPLETED BY Chuck Fuller
TELEPHONE (303)785-2224

OPERATING STATUS

1. Unit Name: Fort St. Vrain
2. Reporting Period: 840301 through 840331
3. Licensed Thermal Power (Mwt): 842
4. Nameplate Rating (Gross MWe): 342
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
None

NOTES

9. Power Level To Which Restricted, If Any (Net MWe): 280
10. Reasons for Restrictions, If Any: Per commitment to the NRC, long term operation above 85% reactor power is pending completion of B-0 Startup Testing.

	This Month	Year to Date	Cumulative
11. Hours in Reporting Period	744	2,184	41,665
12. Number of Hours Reactor Was Critical	0.0	468.0	26,295.3
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator On-Line	0.0	446.6	18,249.8
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	0.0	240,818.8	9,762,136.4
17. Gross Electrical Energy Generated (MWH)	0	77,412	3,230,862
18. Net Electrical Energy Generated (MWH)	-1,919	67,431	2,938,961
19. Unit Service Factor	0.0	20.4	43.8
20. Unit Availability Factor	0.0	20.4	43.8
21. Unit Capacity Factor (Using MDC Net)	0.0	9.4	21.4
22. Unit Capacity Factor (Using DER Net)	0.0	9.4	21.4
23. Unit Forced Outage Rate	0.0	1.5	38.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Refueling 4-1-84/0000 through 5-2-84/2130, 765.5 hours.

25. If Shut Down at End of Report Period, Estimated Date of Startup: 5-2-84

26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY

INITIAL ELECTRICITY

COMMERCIAL OPERATION

Forecast

Achieved

N/A

N/A

N/A

N/A

N/A

N/A

AVERAGE DAILY UNIT POWER LEVEL

TSP-3
Attachment-3A
Issue 2
Page 1 of 1

Docket No. 50-267Unit Fort St. VrainDate April 6, 1984Completed By Chuck FullerTelephone (303)785-2224Month March, 1984DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1	<u>0.0</u>
2	<u>0.0</u>
3	<u>0.0</u>
4	<u>0.0</u>
5	<u>0.0</u>
6	<u>0.0</u>
7	<u>0.0</u>
8	<u>0.0</u>
9	<u>0.0</u>
10	<u>0.0</u>
11	<u>0.0</u>
12	<u>0.0</u>
13	<u>0.0</u>
14	<u>0.0</u>
15	<u>0.0</u>
16	<u>0.0</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17	<u>0.0</u>
18	<u>0.0</u>
19	<u>0.0</u>
20	<u>0.0</u>
21	<u>0.0</u>
22	<u>0.0</u>
23	<u>0.0</u>
24	<u>0.0</u>
25	<u>0.0</u>
26	<u>0.0</u>
27	<u>0.0</u>
28	<u>0.0</u>
29	<u>0.0</u>
30	<u>0.0</u>
31	<u>0.0</u>

*Generator on line but no net generation.

50-267

UNIT NAME Fort St. Vrain

DATE April 6, 1984

COMPLETED BY Chuck Fuller

TELEPHONE (303)785-2224

REPORT MONTH March, 1984

NO.	DATE	TYPE	DURATION	REASON	METHOD OF SHUTTING DOWN REACTOR	LER #	SYSTEM CODE	COMPONENT CODE	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
84- 002	840301	S	744.0	C	2	N/A	ZZZ	ZZZZZZ	Refueling, turbine overhaul, "A" helium circulator changeout, routine corrective and preventive maintenance.

REFUELING INFORMATION

1. Name of Facility	Fort St. Vrain Unit No. 1	
2. Scheduled date for next refueling shutdown.	3rd Refueling: currently underway	4th Refueling: Feb. 1, 1986
3. Scheduled date for restart following refueling.	May 2, 1984, 2130 hours	May 1, 1986
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?	No. Amendment No. 40 dated 3-2-84, approved the use of H-451 graphite.	None Expected
If answer is yes, what, in general, will these be?	-----	-----
If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Reference 10 CFR Section 50.59)?	-----	No
If no such review has taken place, when is it scheduled?	-----	1985
5. Scheduled date(s) for submitting proposed licensing action and supporting information.	-----	-----
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.	-----	-----
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.	1482 HTGR fuel elements 251 spent HTGR fuel elements	

REFUELING INFORMATION (CONTINUED)

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.	Capacity is limited in size to about one-third of core (approximately 500 HTGR elements). No change is planned.
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.	1992 under Agreements AT(04-3)-633 and DE-SC07-79ID01370 between Public Service Company of Colorado, and General Atomic Company, and DOE.*

- * The 1992 estimated date is based on the understanding that spent fuel discharged during the term of the Agreements will be stored by DOE at the Idaho Chemical Processing Plant. The storage capacity has evidently been sized to accomodate eight fuel segments. It is estimated that the eighth fuel segment will be discharged in 1992.



Public Service Company of Colorado

16805 WCR 19 1/2, Platteville, Colorado 80651

April 13, 1984
Fort St. Vrain
Unit #1
P-84111

Office of Inspection and Enforcement
ATTN: Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

REFERENCE: Facility Operating License
No. DPR-34

Docket No. 50-267

Dear Sir:

Enclosed please find our Monthly Operations Report for the month of March, 1984.

Very truly yours,

Don Warembourg
Don Warembourg
Manager, Nuclear Production

Enclosure

cc: John T. Collins

DW/djm

IE24
1/1

NOTE: SEND ORIG TO
Region IV