



Northern States Power Company  
Prairie Island Nuclear Generating Plant  
1717 Wakonade Dr. East  
Welch, Minnesota 55089

May 11, 1995

10 CFR Part 50  
Section 50.554(g)

U S Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT  
Docket Nos. 50-282 License Nos. DPR-42  
50-306 DPR-60

Second Ten-year Interval ASME Section XI Inservice Inspection and  
Testing Program Request for Relief #63, Reactor Coolant Pump Casing Welds

The purpose of this letter is to provide a revision to a relief request for the Unit 2 Second Ten-year Interval ASME Section XI Inservice Inspection and Testing Program. The revision is being provided for your information. Since the revision deletes a portion no longer necessary and leaves unchanged a portion previously approved, no review or approval is necessary.

Relief Request No. 63 was originally submitted on December 22, 1983 and approved by the NRC on December 28, 1984 for relief from the examination requirements for reactor coolant pump casement welds. Relief had been requested from ASME Code Section XI requirements B-L-1, Item B 12.10 and B-L-2, Item B 12.20 Code Case N-481 dated March 5, 1990, which has been approved for use by Regulatory Guide 1.147, is applicable as an alternate to the Code required volumetric examination in Category B-L-1, Item B 12.10.

We plan to use the alternate method of inspection described in Code Case N-481 in lieu of the volumetric requirement described in ASME Code Section XI Category B-L-1, Item B 12.10; therefore, the alternate examination, surface examination of the pump casement weld, as discussed in Revision 0 to Relief Request No. 63, will not be done. Relief Request No. 63 has been revised to delete reference to Category B-L-1, Item B 12.10. Relief Request No. 63 also requested relief from examination requirement B-L-2, Item B 12.20, visual examination of pump internal surfaces. Relief Request No. 63 for Category B-L-2, Item B 12.20 remains unchanged.

In this letter we have made an NRC commitment to replace the original Relief Request #63 alternate examination requirements for the reactor coolant pump casement weld with the requirements listed in Code Case N-481.

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NORTHERN STATES POWER COMPANY

Please contact Jack Leveille (612-388-1121, Ext. 4662) if you have any questions related to this letter.

*Michael D. Whalley for.*

Roger O Anderson

Director

Licensing and Management Issues

c: Regional Administrator - Region III, NRC  
Senior Resident Inspector, NRC  
NRR Project Manager, NRC  
J E Silberg

Attachment: 63. Request for Relief (1 page), dated 3/27/95

### 63. Request for Relief

Component or Item	Code Class	Program Table	Code Item	Exam Category
Reactor Coolant Pump Casing	1	12.1	B12.20	B-L-2

#### Code Requirements

Visual Examination of pump internal surfaces.

#### Basis

The pump manufacturer does not require or recommend pump disassembly to perform normal maintenance and inspection due to limited experienced personnel and possible damage or degradation to the pump.

#### Alternative

If maintenance or operational problems are encountered which require the disassembly of the pump, the pump's interior surface will be visually inspected.

#### Schedule for Implementation

December 21, 1983