

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) R. E. Ginna Nuclear Power Plant, Unit No. 1										DOCKET NUMBER (2) 0 5 0 0 0 2 4 4 1 OF 0 2				PAGE 13		
TITLE (4) Inoperable Residual Heat Removal System																
EVENT DATE (6)			LER NUMBER (8)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (9)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
0 3	0 3	8 4	8 4	0 0 2	0 0	0 3	3	0 8 4					0 5 0 0 0			
OPERATING MODE (5)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)														
N		20.402(b)				20.408(a)				80.73(a)(2)(i)				73.71(b)		
POWER LEVEL (10)		20.408(a)(1)(i)				80.38(a)(1)				80.73(a)(2)(v)				73.71(a)		
0 0 1 0		20.408(a)(1)(ii)				80.38(a)(2)				80.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 308A)		
		20.408(a)(1)(iii)				80.73(a)(2)(i)				80.73(a)(2)(vii)(A)						
		20.408(a)(1)(iv)				80.73(a)(2)(ii)				80.73(a)(2)(vii)(B)						
		20.408(a)(1)(v)				80.73(a)(2)(iii)				80.73(a)(2)(ix)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME G. F. Larizza, Operation Manager										TELEPHONE NUMBER						
										AREA CODE 3 1 5 5 2 4 1 4 4 1 6						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC						
X	B1P	-1-210	L12100	Y												
X	B1P	-11S1V	V10185	Y												
SUPPLEMENTAL REPORT EXPECTED (14)																
X YES (If yes, complete EXPECTED SUBMISSION DATE)										NO		EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
														0 5	1 5	8 4

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On March 3, 1984, while cooling down the Reactor Coolant System (RCS) to cold shutdown condition for the Annual Refueling and Maintenance Outage, periodic test PT-2.4.1 "Cold/Refueling Motor Operated Valve Surveillance (RHR System - 700 valves)" was in progress. MOV-700 (RCS Loop A residual heat removal suction stop valve) failed to stroke to the open position when actuated from the Control Room. Following manual unseating of the valve, the valve was retested and stroking times were verified acceptable (timed twice, full cycle.)

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
R. E. Ginna Nuclear Power Plant Unit No. 1	0 5 0 0 0 2 4 4	8 4	- 0 0 2	- 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 368A's) (17)

On March 3, 1984, while cooling down the Reactor Coolant System (RCS) to cold shutdown condition for the Annual Refueling and Maintenance Outage, periodic test PT-2.4.1 "Cold/Refueling Motor Operated Valve Surveillance (RHR System - 700 Valves)" was in progress. MOV-700 (RCS Loop A Residual Heat Removal Suction Stop Valve) failed to stroke to the open position when actuated from the Control Room. Manual unseating was performed by the Surveillance Group Technician and subsequently the valve was tested satisfactorily and stroking times verified within acceptable tolerance.

When the valve was stroked, the Red Light (open indication) actuated, indicating the valve started to move to the open position. Its movement was stopped by the torque switch operation.

Two most probable causes of failure could be either a dry valve stem, or a light torque switch setting. Field technicians involved in PT-2.4.1, reported that MOV-700 had an extremely dry operating shaft. This lack of lubrication probably contributed to MOV-700 failure to stroke to the open position.

The torque switch setting will be verified and increased if necessary. The valve stem and packing will be inspected. The torque switch bypass setpoint will also be verified and corrected if necessary.

These corrective actions will take place later during the outage when the Reactor Cavity will be flooded for Refueling. Periodic lubrication maintenance will be reviewed. A follow-up report will be made at a later date.



ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649-0001

ROGER W. KOBER
VICE PRESIDENT
ELECTRIC & STEAM PRODUCTION

TELEPHONE
AREA CODE 716 546-2700

March 30, 1984

Dr. Thomas E. Murley, Regional Administrator
U.S. Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

Subject: LER 84-002, Inoperable Residual Heat Removal (RHR)
System

R. E. Ginna Nuclear Power Plant, Unit No. 1
Docket No. 50-244

Dear Dr. Murley:

In accordance with 10 CFR 50.73, Licensee Event Report System, item (a)(2)(vii), "any event where a single cause or condition caused at least one independent train or channel to become inoperable in multiple systems or two independent trains or channels to become inoperable in a single system designed to:

- (a) Shutdown the Reactor and maintain it in a safe shutdown condition;
- (b) Remove Residual Heat;
- (c) Mitigate the consequences of an accident."

The attached Licensee Event Report LER 84-002 is hereby submitted.

Very Truly Yours,

Roger W. Kober

Roger W. Kober

GFL/eeg

xc: Document Control Desk (1)

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