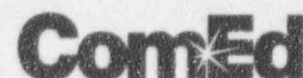


Commonwealth Edison Company
LaSalle Generating Station
2601 North 21st Road
Marseilles, IL 61341-9757
Tel 815-357-6761



May 12, 1995

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Enclosed for your information is the monthly performance report covering
LaSalle County Nuclear Power Station for April, 1995.

A handwritten signature in dark ink, appearing to read "D. J. Ray".

D. J. Ray
Station Manager
LaSalle County Station

DJR/mkl

Enclosure

cc: John B. Martin, Regional Administrator - Region III
NRC Senior Resident Inspector - LaSalle
IL Department of Nuclear Safety - LaSalle
IL Department of Nuclear Safety - Springfield, IL
NRR Project Manager - Washington, D.C.
GE Representative - LaSalle
Regulatory Assurance Supervisor - LaSalle
Licensing Operations Director - Downers Grove
Nuclear Fuel Services Manager - General Office
Off-Site Safety Review Senior Participant - Downers Grove
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Central File

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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

April 1995

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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(UNIT 1)

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1. Main Steam Safety Relief Valve Operations
2. Major Changes to Radioactive Waste Treatment System
3. Static O-Ring Failures
4. Off-Site Dose Calculation Manual Changes

I INTRODUCTION (UNIT 1)

The LaSalle County Nuclear Power Station is a two-Unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois river for make-up and blowdown. The architect-engineer was Sargent and Lundy and the contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815) 357-6761, extension 2056.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical, Generator on-line at 1140 Mwe.
	0010	Reduced power level to 1040 Mwe to transfer the Heater Drain pumps.
	0400	Increased power level to 1140 Mwe.
6	2330	Reduced power level to 1050 Mwe to transfer the Heater Drain and Condensate pumps.
7	0500	Increased power level to 1140 Mwe.
12	2300	Reduced power level to 850 Mwe to perform Main Steam Isolation Valve limit switch testing.
13	0700	Increased power level to 1130 Mwe.
21	1300	Reduced power level to 1025 Mwe to take the 2nd stage Moisture Separator Reheater off-line and for the performance of a rod set.
22	0800	Increased power level to 1140 Mwe.
30	2400	Reactor critical, Generator on-line at 1130 Mwe.

B. AMENDMENTS TO THE FACILITY OR TECHNICAL SPECIFICATION

On April 13, 1995, Amendment 103 was issued to License NPF-11 (Unit 1). This amendment implemented the General Electric Average Power Range Monitor scram/rod block Technical Specifications (ARTS) analysis for thermal limits monitoring.

C. SUBMITTED LICENSEE EVENT REPORTS (Unit 1)

<u>LER No.</u>	<u>Occurrence Date</u>	<u>Description</u>
95-009	03/15/95	Discrepancies were identified between the Updated Final Safety Analysis Report (UFSAR) description of flood protection features and the actual plant design.

D. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 1)
2. Average Daily Unit Power Level (See Table 2)
3. Unit Shutdowns and Significant Power Reductions (See Table 3)

E. UNIQUE REPORTING REQUIREMENTS (UNIT 1)

1. Safety Relief Valve Operations
(None)
2. Major Changes to Radioactive Waste Treatment Systems
(None)
3. Static O-Ring Failures
(None)
4. Changes to the Off-Site Dose Calculation Manual
(None)

TABLE 1
D.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE May 11, 1995
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD:	April 1995	GROSS HOURS IN REPORTING PERIOD	719
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):	3,323	MAX DEPEND CAPACITY (MWe-Net):	1,036
		DESIGN ELECTRICAL RATING (MWe-N	1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):

4. REASONS FOR RESTRICTION (IF ANY):

	REPORTING PERIOD DATA		
	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	719.0	2,748.7	69,388.6
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. GENERATOR ON-LINE TIME (HOURS)	719.0	2,738.7	67,849.9
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MWh _t)	2,376,301	8,913,066	201,599,891
10. ELECTRICAL ENERGY GENERATED (MWe _t -Gross)	809,583	3,033,977	67,424,868
11. ELECTRICAL ENERGY GENERATED (MWe _t -Net)	788,528	2,947,590	64,716,734
12. REACTOR SERVICE FACTOR (%)	100.0	95.5	69.9
13. REACTOR AVAILABILITY FACTOR (%)	100.0	95.5	71.5
14. UNIT SERVICE FACTOR (%)	100.0	95.1	68.3
15. UNIT AVAILABILITY FACTOR (%)	100.0	95.1	68.3
16. UNIT CAPACITY FACTOR (USING MDC) (%)	105.9	98.8	62.9
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	101.7	95.0	60.4
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	4.9	8.1

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

TABLE 2
D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE May 11, 1995
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: April 1995

DAY	POWER	DAY	POWER
1	1,099	17	1,100
2	1,055	18	1,101
3	1,102	19	1,101
4	1,101	20	1,100
5	1,101	21	1,066
6	1,099	22	1,103
7	1,095	23	1,103
8	1,099	24	1,101
9	1,100	25	1,104
10	1,104	26	1,099
11	1,101	27	1,096
12	1,101	28	1,094
13	1,050	29	1,091
14	1,100	30	1,089
15	1,101	31	
16	1,101		

TABLE 3

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(UNIT 1)

<u>YEARLY SEQUENTIAL NUMBER</u>	<u>DATE (YYMMDD)</u>	<u>TYPE F: FORCED S: SCHEDULED</u>	<u>DURATION (HOURS)</u>	<u>REASON</u>	<u>METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER</u>	<u>CORRECTIVE ACTIONS/COMMENTS (LER # if applicable)</u>
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(None)

SUMMARY OF OPERATION:

The unit remained on-line at high power throughout the month. Several minor power reductions were required due to maintenance and surveillance activities.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

April 1995

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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D. DATA TABULATIONS

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

E. UNIQUE REPORTING REQUIREMENTS

1. Main Steam Safety Relief Valve Operations
2. Major Changes to Radioactive Waste Treatment System
3. Static O-Ring Failures
4. Off-Site Dose Calculation Manual Changes

I. INTRODUCTION (UNIT 2)

The LaSalle County Nuclear Power Station is a two-Unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois river for make-up and blowdown. The architect-engineer was Sargent and Lundy and the contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on October 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2056.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor sub-critical, Generator off-line. Refuel outage (L2R06) in progress.
30	2400	Reactor sub-critical, Generator off-line. Refuel outage (L2R06) in progress.

B. AMENDMENTS TO THE FACILITY OR TECHNICAL SPECIFICATION

On April 13, 1995, Amendment 88 was issued to License NPF-18 (Unit 2). This amendment implemented the General Electric Average Power Range Monitor scram/rod block Technical Specifications (ARTS) analysis for thermal limits monitoring.

On April 25, 1995, Amendment 89 was issued to license NPF-18 (Unit 2). This Technical Specification change revised the Safety Relief Valve upper setpoint tolerance from +1% to +3%.

C. SUBMITTED LICENSEE EVENT REPORTS (Unit 2)

<u>LER No.</u>	<u>Occurrence Date</u>	<u>Description</u>
95-006	03/20/95	During the performance of the local leak rate testing on the 2RE024 and 2RE025 valves it was found that the leakage rate of the valves was too excessive to quantify.

D. DATA TABULATIONS (Unit 2)

1. Operating Data Report (See Table 1)
2. Average Daily Unit Power Level (See Table 2)
3. Unit Shutdowns and Significant Power Reductions (See Table 3)

E. UNIQUE REPORTING REQUIREMENTS (UNIT 2)

1. Safety Relief Valve Operations
(See Attachment A)
2. Major Changes to Radioactive Waste Treatment Systems
(None)
3. Static O-Ring Failures
(None)
4. Changes to the Off-Site Dose Calculation Manual
(None)

TABLE 1
D.1 OPERATING DATA REPORT

DOCKET NO. 050-374
UNIT LASALLE TWO
DATE May 11, 1995
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: April 1995 GROSS HOURS IN REPORTING PERIOD: 719

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323 MAX DEPEND CAPACITY (MWe-Net): 1,036
DESIGN ELECTRICAL RATING (MWe-Net): 1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):

4. REASONS FOR RESTRICTION (IF ANY):

	REPORTING PERIOD DATA		
	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	0.0	1,152.6	66,360.6
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9
7. GENERATOR ON-LINE TIME (HOURS)	0.0	1,152.3	65,137.8
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWht)	0	3,731,007	197,482,935
10. ELECTRICAL ENERGY GENERATED (MWhe-Gross)	0	1,281,127	65,969,696
11. ELECTRICAL ENERGY GENERATED (MWhe-Net)	-8,016	1,222,866	63,407,628
12. REACTOR SERVICE FACTOR (%)	0.0	40.0	71.9
13. REACTOR AVAILABILITY FACTOR (%)	0.0	40.0	73.7
14. UNIT SERVICE FACTOR (%)	0.0	40.0	70.6
15. UNIT AVAILABILITY FACTOR (%)	0.0	40.0	70.6
16. UNIT CAPACITY FACTOR (USING MDC) (%)	-1.1	41.0	66.3
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	-1.0	39.4	63.7
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	0.0	10.4

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
05/15/95

TABLE 2
D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-374
UNIT LASALLE TWO
DATE May 11, 1995
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: April 1995

DAY	POWER	DAY	POWER
1	-11	17	-11
2	-11	18	-11
3	-11	19	-11
4	-11	20	-11
5	-11	21	-11
6	-11	22	-11
7	-11	23	-11
8	-11	24	-11
9	-11	25	-11
10	-11	26	-11
11	-11	27	-12
12	-11	28	-12
13	-11	29	-12
14	-11	30	-12
15	-11	31	
16	-11		

TABLE 3

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(UNIT 2)

<u>YEARLY SEQUENTIAL NUMBER</u>	<u>DATE (YYMMDD)</u>	<u>TYPE F: FORCED S: SCHEDULED</u>	<u>DURATION (HOURS)</u>	<u>REASON</u>	<u>METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER</u>	<u>CORRECTIVE ACTIONS/COMMENTS (LER # if applicable)</u>
1	950218	S	719.0	C	2	Refuel outage (L2R06)

SUMMARY OF OPERATION:

The unit was in a scheduled refuel outage for the entire month.

Attachment A

Safety Relief Valve Operations

The following operations took place with the unit shutdown and with the reactor depressurized as part of routine intentional operability testing.

<u>Valve</u>	<u>Date of</u> <u>Actuation</u>	<u>Type of</u> <u>Actuation</u>	<u>Reason for</u> <u>Actuation</u>
2B21F013A	04/06/95	Manual	Operability testing
2B21F013B	04/06/95	Manual	Operability Testing
2B21F013C	04/06/95	Manual	Operability Testing
2B21F013D	04/06/95	Manual	Operability Testing
2B21F013E	04/06/95	Manual	Operability Testing
2B21F013F	04/06/95	Manual	Operability Testing
2B21F013G	04/06/95	Manual	Operability Testing
2B21F013H	04/06/95	Manual	Operability Testing
2B21F013J	04/06/95	Manual	Operability Testing
2B21F013K	04/06/95	Manual	Operability Testing
2B21F013L	04/06/95	Manual	Operability Testing
2B21F013N	04/06/95	Manual	Operability Testing
2B21F013P	04/06/95	Manual	Operability Testing
2B21F013R	04/06/95	Manual	Operability Testing
2B21F013S	04/06/95	Manual	Operability Testing
2B21F013U	04/06/95	Manual	Operability Testing
2B21F013V	04/06/95	Manual	Operability Testing
2B21F013C	04/07/95	Manual	Operability Testing
2B21F013D	04/07/95	Manual	Operability Testing
2B21F013E	04/07/95	Manual	Operability Testing
2B21F013R	04/07/95	Manual	Operability Testing
2B21F013S	04/07/95	Manual	Operability Testing
2B21F013U	04/07/95	Manual	Operability Testing
2B21F013V	04/07/95	Manual	Operability Testing
2B21F013A	04/12/95	Manual	Operability Testing
2B21F013C	04/12/95	Manual	Operability Testing
2B21F013G	04/12/95	Manual	Operability Testing
2B21F013U	04/12/95	Manual	Operability Testing
2B21F013M	04/26/95	Manual	Operability Testing