

OPERATING DATA REPORT

DOCKET NO. 50-266

DATE April 6, 1984

COMPLETED BY C. W. FAY

TELEPHONE 414 277 2811

OPERATING STATUS

1. UNIT NAME: POINT BEACH NUCLEAR PLANT UNIT 1
2. REPORTING PERIOD: MARCH 1984
3. LICENSED THERMAL POWER (MW): 1518.
4. NAMEPLATE RATING (GROSS MWE): 523.8
5. DESIGN ELECTRICAL RATING (NET MWE): 497.
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 519.
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 485.
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:
NOT APPLICABLE

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): NOT APPLICABLE

10. REASONS FOR RESTRICTIONS, (IF ANY): NOT APPLICABLE

	THIS MONTH	YR TO DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744	2,184	117,480
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0.0	0.0	94,078.5
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	625.4
14. HOURS GENERATOR ON LINE	0.0	0.0	91,607.5
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	793.5
16. GROSS THERMAL ENERGY GENERATED (MWH)	0	0	123,535,312
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0	0	41,395,980
18. NET ELECTRICAL ENERGY GENERATED (MWH)	0	0	39,356,940
19. UNIT SERVICE FACTOR	0.0	0.0	78.0
20. UNIT AVAILABILITY FACTOR	0.0	0.0	78.7
21. UNIT CAPACITY FACTOR (USING MDC NET)	0.0	0.0	68.5
22. UNIT CAPACITY FACTOR (USING DER NET)	0.0	0.0	67.4
23. UNIT FORCED OUTAGE RATE	0.0	0.0	2.6
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):	NONE		

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: APRIL 5, 1984

DATA REPORTED AND FACTORS CALCULATED AS REQUESTED IN NRC LETTER DATED SEPTEMBER 22, 1977

8404170194 840331
PDR ADOCK 05000266
R

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-266UNIT NAME Point Beach Unit 1DATE April 6, 1984REPORT MONTH March, 1984COMPLETED BY C. W. FayTELEPHONE 414/277-2811

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting ³ Down Reactor	Licensee Event Report No.	System Code ⁴	Component Code ⁵	Cause and Corrective Action To Prevent Recurrence
3	831001	S	744	C	1	N/A	ZZ	ZZZZZZ	Continuation of 26-week refueling and steam generator replacement outage.

¹ F: Forced
S: Scheduled

² Reason:
A- Equipment Failure (explain)
B- Maintenance or Test
C- Refueling
D- Regulatory Restriction
E- Operator Training & License Exam
F- Administrative
G- Operational Error (explain)
H- Other (explain)

³ Method:
1- Manual
2- Manual Scram
3- Automatic Scram
4- Other (explain)

⁴ Exhibit G-Instructions for Preparation of Data Entry Sheets for LER File (NUREG-0161)

⁵ Exhibit I- Same Source

DOCKET NO. 50-266

UNIT NAME Point Beach Unit 1

DATE April 6, 1984

COMPLETED BY C. W. Fay

TELEPHONE 414/277-2811

AVERAGE DAILY UNIT POWER LEVEL

MONTH March, 1984

<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>
1	<u>-2</u>	11	<u>-2</u>	21	<u>-4</u>
2	<u>-2</u>	12	<u>-2</u>	22	<u>-4</u>
3	<u>-2</u>	13	<u>-2</u>	23	<u>-4</u>
4	<u>-2</u>	14	<u>-3</u>	24	<u>-4</u>
5	<u>-2</u>	15	<u>-3</u>	25	<u>-4</u>
6	<u>-2</u>	16	<u>-3</u>	26	<u>-3</u>
7	<u>-2</u>	17	<u>-4</u>	27	<u>-3</u>
8	<u>-2</u>	18	<u>-4</u>	28	<u>-3</u>
9	<u>-2</u>	19	<u>-4</u>	29	<u>-3</u>
10	<u>-2</u>	20	<u>-4</u>	30	<u>-3</u>
				31	<u>-3</u>

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

Docket No. 50-266
Unit Name Point Beach Unit 1
Date April 6, 1984
Prepared By C. W. Fay
Telephone 414/277-2811

Unit 1 completed weeks 23 through 26 of the refueling and steam generator replacement outage.

An inspection of the reactor vessel outlet nozzle-to-shell welds was performed using remote mechanized ultrasonics. This exam was part of the second ten-year interval first period inservice inspection program requirements. Four indications in the "A" nozzle and seven indications in the "C" nozzle were recorded, sized, and evaluated. On 02/28/84, Southwest Research Institute notified Wisconsin Electric that there was an indication in each of the outlet nozzle welds that exceeded the Code allowable size. The indications are located in the root area of the welds. It is believed that the reflectors are due to entrapped foreign material in the weld and/or adjacent base metal repair area, and have been there since the vessel was fabricated and are not indicative of service-induced flaws. A Licensee Event Report was drafted for submittal to the NRC.

On 02/25/84, a visual inspection of the control rod guide tube split pins revealed that three nuts were missing. The visual inspection was performed after an ultrasonic inspection of the pins identified cracks in 67 of 74 pins (two pins per guide tube). The UT indications on the shank-to-collar region of the CRGT support pins and the three missing support pin nuts and shanks at Point Beach Unit 1 are judged not to be a safety concern due either to CRGT misalignment preventing RCCA insertion or the adverse effects of loose parts on safety systems and components. The long-range solution to this problem is to replace the pins at a future outage when the tooling and parts are available. A Licensee Event Report was drafted for submittal to the NRC.

Minor insulation work on the steam generators was completed and the equipment hatch was reinstalled. The core barrel and lower internals were removed from the reactor vessel for further PaR examination of the cold leg nozzles. When the PaR examination was complete, the core barrel and lower internals were replaced and the core reload began on Monday, 03/05/84.

The reactor was refueled and the upper internals were put back in place. A problem with debris in the upper internals was discovered when difficulty was encountered in latching the control rods. The upper internals were again removed and the debris was cleaned away from them. Debris removal via the "hydrolaser" method was completed on 03/24/84 and the internals were replaced. Thirteen of the 33 full-length rod control cluster guide tube assemblies had one or two particles left in them. On 03/25/84, the control rods were latched. The head was again set in place and tensioned.

On 03/30/84, the pump-up of containment commenced for the integrated leak rate test (ILRT). The temporary structures erected for the steam generator replacement were disassembled and removed along with the Morrison-Knudsen health physics operation being shut down. The containment was completely turned back to Point Beach Nuclear Plant and the only remaining job for Morrison-Knudsen is the steam generator shim gap measurements to be taken during heat-up.

Safety-related maintenance performed during the period included repairing the pressurizer power-operated relief valves, modifying the control room control panels, repairing the pressurizer spray valve, modifying the solenoid valves on the main feed regulating valves, inspecting/repairing the "A" and "B" component cooling pumps, inspecting the containment spray pumps, and repairing the "B" loop RTD bypass manifold outlet isolation valve.

OPERATING DATA REPORT

DOCKET NO. 50-301

DATE April 6, 1984

COMPLETED BY C. W. FAY

TELEPHONE 414 277 2811

OPERATING STATUS

1. UNIT NAME: POINT BEACH NUCLEAR PLANT UNIT 2
2. REPORTING PERIOD: MARCH 1984
3. LICENSED THERMAL POWER (MWT): 1518.
4. NAMEPLATE RATING (GROSS MWE): 523.8
5. DESIGN ELECTRICAL RATING (NET MWE): 497.
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 519.
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 485.
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:
NOT APPLICABLE
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): NOT APPLICABLE
10. REASONS FOR RESTRICTIONS, (IF ANY): NOT APPLICABLE

	THIS MONTH	YR TO DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744	2,184	102,265
12. NUMBER OF HOURS REACTOR WAS CRITICAL	744.0	2,184.0	90,612.2
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	198.3
14. HOURS GENERATOR ON LINE	744.0	2,184.0	89,086.8
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	182.7
16. GROSS THERMAL ENERGY GENERATED (MWH)	1,125,590	3,288,250	124,183,027
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	379,060	1,108,710	42,068,540
18. NET ELECTRICAL ENERGY GENERATED (MWH)	362,985	1,061,615	40,062,215
19. UNIT SERVICE FACTOR	100.0	100.0	87.1
20. UNIT AVAILABILITY FACTOR	100.0	100.0	87.3
21. UNIT CAPACITY FACTOR (USING MDC NET)	100.6	100.2	79.7
22. UNIT CAPACITY FACTOR (USING DER NET)	98.2	97.8	78.8
23. UNIT FORCED OUTAGE RATE	0.0	0.0	1.4
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): None			
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: NOT SHUTDOWN			

DATA REPORTED AND FACTORS CALCULATED AS REQUESTED IN NRC LETTER DATED SEPTEMBER 22, 1977

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH March, 1984DOCKET NO. 50-301UNIT NAME Point Beach Unit 2DATE April 6, 1984COMPLETED BY C. W. FayTELEPHONE 414/277-2811

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report No.	System Code ⁴	Component Code ⁵	Cause and Corrective Action To Prevent Recurrence

¹ F: Forced
S: Scheduled

² Reason:
A- Equipment Failure (explain)
B- Maintenance or Test
C- Refueling
D- Regulatory Restriction
E- Operator Training & License Exam
F- Administrative
G- Operational Error (explain)
H- Other (explain)

³ Method:
1- Manual
2- Manual Scram
3- Automatic Scram
4- Other (explain)

⁴ Exhibit G-Instructions for Preparation of Data Entry Sheets for LER File (NUREG-0161)

⁵ Exhibit I- Same Source

DOCKET NO. 50-301
UNIT NAME Point Beach Unit 2
DATE April 6, 1984
COMPLETED BY C. W. Fay
TELEPHONE 414/277-2811

AVERAGE DAILY UNIT POWER LEVEL

MONTH March, 1984

<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>
1	<u>490</u>	11	<u>488</u>	21	<u>488</u>
2	<u>489</u>	12	<u>489</u>	22	<u>486</u>
3	<u>490</u>	13	<u>490</u>	23	<u>486</u>
4	<u>490</u>	14	<u>488</u>	24	<u>487</u>
5	<u>490</u>	15	<u>490</u>	25	<u>484</u>
6	<u>487</u>	16	<u>490</u>	26	<u>467</u>
7	<u>490</u>	17	<u>488</u>	27	<u>487</u>
8	<u>489</u>	18	<u>489</u>	28	<u>488</u>
9	<u>490</u>	19	<u>489</u>	29	<u>488</u>
10	<u>490</u>	20	<u>489</u>	30	<u>488</u>
				31	<u>488</u>

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

Docket No. 50-301
Unit Name Point Beach Unit 2
Date April 6, 1984
Prepared By C. W. Fay
Telephone 414/277-2811

Unit 2 operated at approximately 486 MWe net throughout the period with no significant load reductions.

Primary-to-secondary leakage remained stable at less than 10 gallons per day.

On 03/26/84, at 0843 hours, Unit 2 reached the 42 billion kilowatt-hour mark. Also, On 03/26/84, the plant circulating water system was removed from the ice-melt mode of operation.

On 03/21/84, the "A" and "D" moisture separator reheaters were isolated because of tube leaks and operation continued with 2 moisture separator reheaters.

The pipe snubber in location ID No. 2HS-M75 as listed in Technical Specification Table 15.3.13-1 was removed prior to obtaining a Technical Specification change to allow its removal. The snubber was reinstalled within 72 hours of the event date to satisfy the Limiting Condition of Operation specified in Technical Specification 15.3.13.2. The snubber had been removed as part of a major modification to the pressurizer relief valve piping supports based on reanalysis required by NUREG-0737. At no time was the system in an unsafe or unanalyzed condition. A Technical Specification change to delete the snubber has been initiated. A Licensee Event Report has been submitted to the NRC.

Safety-related maintenance included repairing the "B" steam generator pressure bistable 2PC-483A and repairing a snubber on the safety injection line.



Wisconsin Electric POWER COMPANY
231 W. MICHIGAN, P.O. BOX 2046, MILWAUKEE, WI 53201

April 10, 1984

Director of Regulatory Operations
U. S. NUCLEAR REGULATORY COMMISSION
Washington, D. C. 20555

Gentlemen:

MONTHLY OPERATING REPORTS
POINT BEACH NUCLEAR PLANT

Attached are monthly operating reports for Units 1 and
2, Point Beach Nuclear Plant, for the calendar month of March
1984.

Very truly yours,

Vice President-Nuclear Power

C. W. Fay

Attachments

Copies to J. G. Keppler - NRC, Region III
NRC Resident Inspector
C. F. Riederer - PSCW

50-266
50-301

IE-24
11